

Georgia Power Company
40 Inverness Center Parkway
Post Office Box 1295
Birmingham, Alabama 35201
Telephone 205 877-7279

J. T. Beckham, Jr.
Vice President - Nuclear
Hatch Project



February 6, 1995

Docket No. 50-321

HL-4775

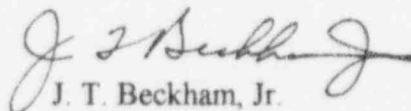
U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Edwin I. Hatch Nuclear Plant - Unit 1
Licensee Event Report
Personnel Error Results in Missed
Technical Specifications Surveillance

Gentlemen:

In accordance with the requirements of 10 CFR 50.73(a)(2)(i), Georgia Power Company is submitting the enclosed Licensee Event Report (LER) concerning a personnel error which resulted in a missed Technical Specification surveillance.

Sincerely,



J. T. Beckham, Jr.

OCV/et

Enclosure: LER 50-321/1995-001

cc: Georgia Power Company
Mr. H. L. Sumner, General Manager - Nuclear Plant
NORMS

U.S. Nuclear Regulatory Commission, Washington, D.C.
Mr. K. Jabbour, Licensing Project Manager - Hatch

U.S. Nuclear Regulatory Commission, Region II
Mr. S. D. Ebner, Regional Administrator
Mr. B. L. Holbrook, Senior Resident Inspector - Hatch

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS
INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD
COMMENTS REGARDING BURDEN ESTIMATE TO THE
INFORMATION AND RECORDS MANAGEMENT BRANCH
(MNNB7714), U.S. NUCLEAR REGULATORY COMMISSION,
WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK
REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND
BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Edwin I. Hatch Nuclear Plant - Unit 1

DOCKET NUMBER (2)

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TITLE (4)

Personnel Error Results in Missed Technical Specifications Surveillance

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME					DOCKET NUMBER(S)																			
									FACILITY NAME																								
0	1	10	9	5		9	5		0	0	1	0	0		0	2	0	6	9	5	0 5 0 0 0												
OPERATING MODE (9)									THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 2. (Check one or more of the following) (11)																								
1									20 402(b)					20 405(c)					50 73(a)(2)(iv)					73.71(b)									
POWER									20 405(a)(1)(i)					50 36(c)(1)					50 73(a)(2)(iv)					73.71(c)									
LEVEL (10)									1 0 0					20 405(a)(1)(ii)					50 36(c)(2)					50 73(a)(2)(vi)					OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
									20 405(a)(1)(iii)					X 50 73(a)(2)(i)					50 73(a)(2)(vii)(A)														
									20 405(a)(1)(iv)					50 73(a)(2)(ii)					50 73(a)(2)(viii)(B)														
									20 405(a)(1)(v)					50 73(a)(2)(iii)					50 73(a)(2)(x)														

LICENSEE CONTACT FOR THIS LER (12)

NAME _____

Steven B. Tipps, Nuclear Safety & Compliance Manager, Hatch

TELEPHONE NUMBER (include area code)

AREA CODE

9	1	2	3	6	7	-	7	8	5	1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	X	NO	SUBMISSION DATE (15)			
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ABSTRACT (limit to 1400 spaces, i.e. approximately 15 single-space typewritten lines) (16)

On 1/10/95, Unit 1 was in the Run mode at a power level of 2436 CMWT (100% rated thermal power). At that time, personnel determined that a surveillance required by the Unit 1 Technical Specifications had been missed in May 1994. Specifically, the inservice test of Residual Heat Removal system pumps 1E11-C002B and 1E11-C002D, required by Unit 1 Technical Specifications section 4.6.K.1, had not been performed between 1/7/94 and 6/21/94. The surveillance is required to be performed once every three months; therefore, it should have been performed no later than 5/3/94 (three months from 1/7/94 plus the 25% grace period allowed by the Technical Specifications). The surveillance task sheet for the inservice test had been erroneously signed off as complete on 3/29/94 and the surveillance tracking system computer data base updated accordingly. Thus, the surveillance was not listed by the computer in April 1994 as potentially overdue and was not scheduled to be performed again until June 1994. Consequently, the inservice test on pumps 1E11-C002B and 1E11-C002D was not performed by 5/3/94; it was performed successfully on 6/21/94.

The cause of this event was personnel error. The Operations Shift Supervisor, believing the inservice test had been performed, (in addition to the operability test which was actually run) signed a surveillance task sheet indicating the test had been completed. Both tests are contained in the same procedure. The Shift Supervisor was counseled regarding his error and its consequences.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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Edwin I. Hatch Nuclear Plant - Unit 1

05000321

YEAR

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YEARREVISION
NUMBER

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TEXT (If more space is required, use additional copies of NRC Form 366A)(17)

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor

Energy Industry Identification System codes are identified in the text as (EIIIS Code XX).

DESCRIPTION OF EVENT

On 1/10/95, Unit 1 was in the Run mode at a power level of 2436 CMWT (100% rated thermal power). At that time, plant Operations personnel, after unsuccessfully searching for data to fill a gap in inservice test results for two pumps, determined that a surveillance required by the Unit 1 Technical Specifications had been missed in May 1994. Specifically, the inservice test of Residual Heat Removal (RHR, EIIIS Code BO) system pumps 1E11-C002B and 1E11-C002D, required by Unit 1 Technical Specifications section 4.6 K.1 to be performed once every three months, had not been performed at the required frequency.

The inservice test on RHR system pumps 1E11-C002B and 1E11-C002D had been performed on 1/7/94 and therefore should have been performed no later than 5/3/94. This date is three months from 1/7/94 plus the 25% grace period allowed by the Technical Specifications. The surveillance task sheet for the inservice test had been erroneously signed-off as complete on 3/29/94. However, only a pump operability test was performed on 3/29/94. The operability test does not collect all the data required by the inservice test. Additionally, different data analysis is required for the inservice test. Thus, the task sheet for the inservice test should not have been signed-off as complete.

The surveillance tracking system computer data base was updated with the incorrect information regarding inservice test performance for RHR system pumps 1E11-C002B and 1E11-C002D. This resulted in the surveillance not being listed by the tracking system computer as potentially overdue in April 1994. Additionally, based upon the incorrect performance date of 3/29/94, the inservice test was not scheduled by the computer to be performed again until June 1994. Consequently, the inservice test on RHR system pumps 1E11-C002B and 1E11-C002D was not performed by 5/3/94 as required by the Technical Specifications. It was performed successfully on 6/21/94.

CAUSE OF EVENT

The cause of this event was personnel error. The Unit 1 Operations Shift Supervisor, believing the inservice test had been performed, erroneously signed-off as complete the surveillance task sheet for the inservice test of RHR system pumps 1E11-C002B and 1E11-C002D on 3/29/94 when, in fact, the inservice test had not been performed. Only the pump operability test was performed on 3/29/94. Both tests are contained in the same procedure. The incorrect inservice test performance

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information from the task sheet was input into the surveillance tracking system computer; consequently, the computer did not identify the inservice test as potentially overdue in April 1994 and the test was not scheduled to be performed again until June 1994.

REPORT ABILITY ANALYSIS AND SAFETY ASSESSMENT

This report is required by 10 CFR 50.73 (a)(2)(i) because a condition existed which was prohibited by the plant's Technical Specifications. Specifically, the inservice test of RHR system pumps 1E11-C002B and 1E11-C002D was not performed at the frequency required by Unit 1 Technical Specifications section 4.6 K.1. The inservice test is required to be performed once every three months, plus a 25% grace period; however, over five months elapsed between consecutive test performances.

The Residual Heat Removal system consists of two independent loops, each containing two pumps. Surveillances, such as pump operability and inservice tests, are performed periodically as prescribed by the Unit 1 Technical Specifications to ensure the system is capable of performing its design functions: these functions include decay heat removal and low pressure coolant injection. A pump operability test checks pump flow rate and discharge pressure to ensure it can reach design rated flow at a given discharge pressure. An inservice test checks pump flow rate, discharge pressure, and vibration and compares its operation to a baseline condition established when the pump was known to be in good working order. The purpose of inservice testing is to detect pump degradation so corrective actions can be taken before the pump degrades below its ability to perform its design function.

In this event, a required inservice test on two of the four RHR system pumps was missed. Inservice tests were performed on pumps 1E11-C002B and 1E11-C002D on 1/7/94 and on 6/21/94, but not in May 1994 as required by the Unit 1 Technical Specifications. An operability test was performed successfully on these two pumps on 3/29/94. A review of the test data from the 1/7/94 and 6/21/94 inservice tests shows the performance of the pumps had not degraded during the period between tests. Additionally, the two pumps passed their operability tests on 3/29/94. It is reasonable therefore to conclude that the two pumps were fully capable of performing their design functions during the entire period between 1/7/94 and 6/21/94. Furthermore, it may be concluded that the pumps were not degrading during this period and that no adverse pump performance trends went undetected as a result of this event.

Based upon the preceding discussion, it is concluded that this event had no adverse impact on nuclear safety. This analysis is applicable to all power levels and operating modes.

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CORRECTIVE ACTIONS

The Operations Shift Supervisor was counseled on the importance of paying attention to detail, especially regarding the performance of Tech Spec surveillances. Furthermore, the consequences, and potential consequences, of his actions were pointed out, i.e., potentially missing indications of pump degradation, and the need to generate a License Event Report.

ADDITIONAL INFORMATION

No systems other than those mentioned in this report were effected by this event.

No failed components caused or resulted from this event.

Previous similar events reported in the last two years in which surveillances required by the plant's Technical Specifications were missed due to personnel error were reported in the following Licensee Event Reports:

50-321/1993-003, dated 5/10/93
50-321/1994-004, dated 5/9/94
50-321/1994-015, dated 12/19/94
50-366/1994-006, dated 5/24/94

Corrective actions for the previous events could not have prevented this event because the previous events involved different personnel, different surveillances, and different errors, i.e., the prior errors did not involve personnel erroneously placing a completion date on the surveillance task sheet. Previous actions addressed the personnel and the errors which caused the particular events. Because these personnel and errors were different, the corrective actions taken could not reasonably be expected to have prevented this individual from making the specific error which led to the missed inservice test on RHR system pumps 1E11-C002B and 1E11-C002D.