

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE						COMP. SUBCODE		VALVE SUBCODE			
C	E	E	E	I	N	S	T	R	U	C	Z						
9	10	11	12	13	14	15	16	17	18	19	20						
EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.									
8	3	0	8	2	0	3	L	0									
21	22	23	24	25	26	27	28	29	30	31	32						
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
A	Z	Z	Z	0	0	0	0	Y	N	N	W	2	9	0			
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47			

FACILITY STATUS							% POWER						OTHER STATUS									METHOD OF DISCOVERY								DISCOVERY DESCRIPTION																				
E (28)							099 (29)						NA									A (31)								Operator Observation																				
ACTIVITY CONTENT RELEASED OF RELEASE													AMOUNT OF ACTIVITY													LOCATION OF RELEASE																								
Z (33) Z (34)													NA													NA																								
PERSONNEL EXPOSURES NUMBER TYPE													DESCRIPTION																																					
000 (37) Z (38)													NA																																					
PERSONNEL INJURIES NUMBER													DESCRIPTION																																					
000 (40)													NA																																					
LOSS OF OR DAMAGE TO FACILITY TYPE													DESCRIPTION													8310250291 831017 PDR ADDCK 05000366 S PDR																								
Z (42)													NA																																					
PUBLICITY ISSUED													DESCRIPTION													NRC USE ONLY																								
N (44)													NA																																					

PHONE: (912) 367-7851

NARRATIVE REPORT
FOR LER 50-366/1983-082

LICENSEE : GEORGIA POWER COMPANY
FACILITY NAME : EDWIN I. HATCH
DOCKET NUMBER : 50-366

Tech. Specs. section(s) which requires report:

This 30-day LER is required by Tech. Specs. section 6.9.1.9.b due to the event's showing that the unit was not meeting the requirements of Tech. Specs. section 3.7.3.

Plant conditions at the time of the event(s):

On 09/22/83, the plant was in steady-state operation at 2414 MWt (approximately 99% reactor power).

On 09/25/83 the plant was in start-up following a reactor scram on 09/23/83 with the reactor at 626 MWt (approximately 26% reactor power).

Detailed description of the event(s):

On 09/22/83, during a routine control panel check, operating personnel determined that the RCIC flow controller milliamp control station demand indication (2E51-R612) was less than 100% (i.e., as found indication 95% demand).

On 09/25/83, during performance of the "RCIC PUMP OPERABILITY" procedure (HNP-2-3405) RCIC failed to reach rated flow and pressure. Note, this test was being performed to prove RCIC operable following corrective action for the first event.

Consequences of the event(s):

Plant conditions were not affected by this event. The health and safety of the public were not affected by this event.

Status of redundant or backup subsystems and/or systems:

HPCI remained operable at the time of this event.

Justification for continued operation:

Plant operations continued under a 14-day LCO as allowed by Tech. Specs. section 3.7.3, ACTION a. This LCO started upon the first failure and was not cleared until the second event was corrected and RCIC proven operable.

If repetitive, number of previous LER:

The first event is non-repetitive; however, the second event is repetitive as last reported by LER 50-366/1983-055.

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Impact to other systems and/or Unit:

This event had no impact on any other system nor to the other unit.

Cause(s) of the event(s):

The cause of the inaccurate demand indication was due to setpoint drift. An investigation on the first event revealed that the high current limiter on the RCIC control amplifier (2E51-K616) was out of adjustment.

An investigation determined that RCIC flow controller's ramp generator signal converter 2E51-R751-2 had failed, preventing RCIC from reaching rated flow and pressure on 09/25/83.

Immediate Corrective Action:

The high current limiter on RCIC control amplifier was adjusted per the "GENERAL ELECTRIC 543-03 CONTROLLER AMPLIFIER" procedure (HWP-2-5258) to correct the problem observed in the first event.

The RCIC RAMP GENERATOR SIGNAL CONVERTER (2E51-R751-2) was replaced and calibrated per "WOODARD EGM EGR HPCI AND RCIC TURBINE GOVERNOR" procedure (HNP-2-5287). The "RCIC PUMP OPERABILITY" procedure was run successfully on 09/27/83, and RCIC was returned to service.

Supplemental Corrective Action:

No supplemental corrective action is required.

Scheduled (future) corrective action:

No scheduled corrective action is required.

Action to prevent recurrence (if different from corrective actions):

N/A

Georgia Power Company
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Baxley, Georgia 31513
Telephone 912 367-7781
912 537-9444



Georgia Power

Edwin I. Hatch Nuclear Plant

83 OCT 19 A9:13

October 17, 1983
GM-83-993

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
Suite 3100
101 Marietta Street
Atlanta, Georgia 30303

ATTENTION: Mr. James P. O'Reilly

Attached is Licensee Event Report No. 50-366/1983-082. This report is required by Hatch Unit 2 Technical Specifications Section 6.9.1.9.b.

Lewis Sumner
H. C. Nix

for General Manager

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HCN/SBT/djs

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