

LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 P A T M I L 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 31 CAT 32 33

CON'T

REPORT SOURCE 7 8 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
L 6 0 5 0 0 0 2 8 9 7 0 8 2 5 8 3 8 0 9 3 0 0 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

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02	A no sample flow condition through RM-L-7 was observed. The Industrial Waste
03	Filter and Treatment system (IWFS, IWTS) were not in operation. No discharges
04	were in progress. Public health and safety were unaffected. This is considered
05	reportable per T.S. 6.9.2.B.2.

06

07 _____

08 _____ 80

DATA WORD

0 9

SYSTEM CODE

CAUSE CODE

CAUSE SUBCODE

COMP. SUBCODE

VALVE SUBCODE

11 12 13 14 15 16 19 20

(17) LER/RO REPORT NUMBER [EVENT YEAR] [REPORT NO.] [CODE] [TYPE] [NO.]
 [8] [3] [0] [2] [1] [0] [3] [L] [0]
 ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD FORM SUB PRIME COMP SUPPLIER COMPONENT MANUFACTURER
 [E] [18] [C] [19] [Z] [20] [Z] [21] [0] [0] [0] [0] [Y] [23] [N] [24] [Z] [25] [Z] [9] [9] [9]
 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The RM-L7 outlet valve was apparently clogged due to corrosion in the line.

1 1 Normally less than 1/2 turn of this valve (SR-V-46B) provides the necessary 2 gpm

1 2 flow. Flow was restored within 8 hours. Piping will be replaced with stainless

1 3 steel to prevent recurrence. Both inlet and outlet valves have been locked open.

14 _____

1 5 X 28 0 0 0 29 NRC Order B 31 Operator Observation

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)
6 2 33 2 34 N/A N/A

PERSONNEL EXPOSURES										
NUMBER			TYPE	DESCRIPTION						
1	7	0	0	0	(37)	Z	(38)	N/A		

PERSONNEL INJURIES		DESCRIPTION	
NUMBER			
0	0	(40)	N/A

LOSS OF OR DAMAGE TO FACILITY		(4)
TYPE	DESCRIPTION	
1	2 (4)	N/A

8310250253 830930
PDR ADOCK 05000289
S PDR N/A

NAME OF PREPARER R. A. Szczech PHONE (717) 948-8833

RM-L-7 (MECHANICAL DRAFT EFFLUENT) NO FLOW CONDITIONI. CURRENT ACTIVITIES AT THE TIME OF OCCURRENCE

TMI Unit I was in a cold shutdown condition, RCS pressure was 310 psig and Tcold was 130°F with Reactor Coolant Pump 1B (RC-P-1B) and Decay Heat Pump 1A (DH-P-1A) in operation. The Industrial Waste Filter System (IWFS) and Industrial Waste Treatment System (IWTS) was shutdown. No releases were in progress through RM-L-6 (Radioactive Waste Water Discharge).

II. CIRCUMSTANCES LEADING TO THE OCCURRENCE

At 0930 on August 25, 1983 the outbuildings auxiliary operator proceeded to the Radiation Monitor Pit Building to observe the flow rate on RM-L-7 following a report by a Start Up Engineer that no flow existed on RM-L-7.

III. DESCRIPTION

Following the no flow condition the outlet valve for RM-L-7 (SR-V-46B) was inspected and was thought to be throttled closed. SR-V-46B was then opened and flow rate established at 2gpm. Flow subsequently was observed to decrease and the operator fully opened SR-V-46B. Flow was observed to be 3.5 gpm with subsequent readings indicating flow is stable at approximately 3.5 gpm.

It is believed that at the time of the original check of SR-V-46B the valve was in fact slightly throttled open to permit 2 gpm flow. (This is due to the fact that less than 1/2 turn open is required to provide 2 gpm flow rate through RM-L-7.) However, the line and valve seat were most likely clogged, eliminating flow, causing the operator to believe the valve was closed. With no sample flow, RM-L-7 was inoperable. Flow was restored in less than 8 hours. This event is considered reportable in accordance with Technical Specification 6.9.2.B.2.

IV. RESULTANT EVENTS

None. No radioactive liquid releases were in progress.

V. PREVIOUS EVENTS OF A SIMILAR NATURE

Reference LER 83-017, RM-L-7 was discovered with no flow.

VI. ROOT CAUSE

The use of carbon steel for this piping system has led to severe corrosion and flow problems.

VII. IMMEDIATE CORRECTIVE ACTION

SR-V-46B (outlet valve) has been required to be in the fully open position. This causes the RM-L-7 flow rate to vary slightly about 3.5 gpm. The inlet and outlet valves are locked open with the bypass valve locked closed.

VIII. LONG TERM CORRECTIVE ACTION

- RM-L-7 piping will be replaced with stainless steel to correct the clogging and flow rate problem.

IX. COMPONENT FAILURE DATA

No components failed. RM-L-7 was degraded due to clogging of SR-V-46B (discharge valve).



GPU Nuclear Corporation
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717 944-7621
TELEX 84-2386
Writer's Direct Dial Number:

September 30, 1983
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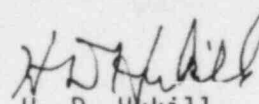
Dr. Thomas E. Murley
Region I, Regional Administrator
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Dear Sir:

Three Mile Island Nuclear Station, Unit 1 (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
LER 83-021/03L-0

This letter transmits Licensee Event Report 83-021/03L-0 concerning station liquid effluent monitor inoperability. Public health and safety were unaffected. This report is being submitted on this date rather than September 26, 1983, per discussion with R. Conte.

Sincerely,


H. D. Hukill
Director, TMI-1

HDH:RAS:vjf

Attachments

cc: J. Van Vliet
Document Management Branch

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