

September 26, 1983

Dr. Thomas E. Murley  
Regional Administrator  
United States Nuclear Regulatory Commission  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

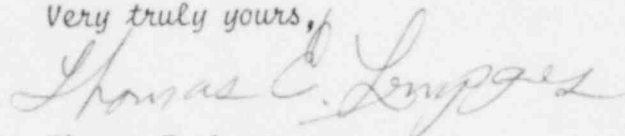
Re: Docket No. 50-220  
LER 83-23

Dear Dr. Murley,

On August 9, 1983, the circuit breaker for core spray pump suction valve 81-21 was taken out of service to perform preventative maintenance while leaving the valve in the open position. Not having motive power available was initially thought to be contrary to primary containment isolation criteria and was promptly reported in accordance with 6.9.2.c(6) of the Nine Mile Point Unit 1 Technical Specifications. The basis for the initial determination was specification 3.3.4.b, which requires any line with an inoperable isolation valve to have at least one valve in the line in the isolated condition. Further investigation, however, showed that the valve in question is erroneously identified as a containment isolation valve on Table 3.3.4 of the Technical Specifications. This is substantiated by a statement on page VI-55 of the FSAR (this section being the basis for Technical Specification 3.3.4) that states, "The engineered safeguards systems which may be required to operate following an accident have no specific isolation requirements.... Valves are provided in the lines from the suppression chamber, and those into the drywell only provide system isolation for maintenance or testing." Therefore, the event did not violate the intent of the Technical Specifications since the FSAR criteria was met. Consequently, the prompt report for LER 83-23 is hereby nullified, and a 14 day followup report will not be forthcoming.

Discrepancies in the existing Technical Specification Table 3.3.4 have been previously discussed with the NRC, and a program is currently underway to clarify the existing table with a Technical Specification Amendment Request.

Very truly yours,



Thomas E. Lempges  
Vice President  
Nuclear Generation

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