

CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)0 1 P A S E S I 2 0 0 0 0 0 0 0 0 0 0 0 3 4 1 1 1 1 4 5
7 8 9 14 15 25 26 30 37 38

CONT

0 1 REPORT SOURCE L 6 0 5 0 0 0 3 8 7 7 0 8 2 8 8 3 8 0 9 2 7 8 3 9
7 8 60 61 66 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During a post-scrum vessel level fluctuation on 8/28/83, RCIC initiated on low RPV
0 3 level, and the turbine tripped 3 seconds later on electrical overspeed. Operations
0 4 personnel established manual control of the system and adjusted turbine speed to
0 5 maintain proper vessel level. RCIC is not an Emergency Core Cooling System and is
0 6 not required for safe shutdown following a scram. There were no adverse effects to
0 7 public health and safety, nor was the unit's safe shutdown capability affected.

0 8

0 9 C E 11 B 12 A 13 V A L V E X 14 H 15 G 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20

17 LER/RO REPORT NUMBER 8 3 21 22
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
F 18 X 19 Z 20 Z 21 0 C 0 0 Y 23 Y 24 N 25 T 1 4 7 26
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The overspeed trip was caused by the slow response of the governor valve during
1 1 system start. The governor valve linkage travel was reduced by one-quarter inch and
1 2 the system successfully retested. RCIC performance will be reviewed and the quick
1 3 start surveillance testing frequency adjusted accordingly to assure proper system
1 4 operation. Installation of a bypass line around the RCIC stm. supply vlv. planned.

1 5 G 28 0 0 0 0 29 NA 30 A 31 Operator observation 32
7 8 9 10 11 12 13 44 45 46 601 6 Z 33 Z 34 NA 35 NA 36
7 8 9 10 11 44 45 601 7 0 0 0 37 Z 38 NA 39
7 8 9 10 11 12 13 601 8 0 0 0 40 NA 41
7 8 9 10 11 12 601 9 Z 42 NA 43
7 8 9 10 602 0 N 44 NA 45
7 8 9 10 602 1 N 46 NA 47
7 8 9 10 602 2 N 47 NA 48
7 8 9 10 60

NAME OF PERSONALITY J.J. Graham

PHONE: (717) 542-2181

ATTACHMENT

LER # 83-120/03L-0

Pennsylvania Power & Light Company
Susquehanna Steam Electric Station
Docket Number: 50-387

During a post-scram Reactor Pressure Vessel (RPV) water level fluctuation on August 28, 1983, the Reactor Coolant Isolation Cooling (RCIC) system automatically initiated when the low water level setpoint was reached. The system turbine tripped on electrical overspeed three seconds after initiation. Operations personnel restored the system to service manually.

The scram was initiated by spurious pressure signals, resulting in a Main Steam Line isolation and the predicted subsequent vessel level fluctuation. There were no adverse consequences as a result of the turbine trip.

Investigation determined that the cause of the trip was due to governor valve response, i.e., the governor valve did not respond as desired during the turbine start sequence, not closing in time to prevent the electrical overspeed trip. As corrective action, the governor valve linkage was adjusted, reducing the 'travel' to 5/8", to ensure closure before an overspeed condition can occur. Additional corrective action included: A change-out of the turbine control oil and installation of replacement control oil filters; installation of a blowdown drain in the steam supply line; and verification of steam line drain pot level indication. Installation of a bypass line around the RCIC Turbine Steam Supply Valve is planned.

RCIC performance is under review and, if necessary, the quick start test frequency will be adjusted accordingly.

The system was tested satisfactorily and returned to service.



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

September 27, 1983

Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 83-120/03L-0
ER 100450 FILE 841-23
PLA-1878

Docket No. 50-387
License No. NPF-14

Dear Dr. Murley:

Attached is Licensee Event Report No. 83-120/03L-0. This event was determined to be reportable per Technical Specification 6.9.1.9.b, in that during a post-scrum Reactor Pressure Vessel (RPV) water level fluctuation, the Reactor Core Isolation Cooling (RCIC) system turbine was automatically initiated and subsequently tripped on overspeed. The turbine governor valve linkage was adjusted and the system returned to service. The system is not an Emergency Core Cooling System. There were no adverse consequences due to the turbine trip nor was the units safe shutdown capability affected.

Related Licensee Event Reports are no's. 83-051/03L-0 and 83-103/03L-0.

H.W. Keiser
Superintendent of Plant-Susquehanna

JJG/pjg

Attachment

cc: G.G. Rhoads
Resident Inspector
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Washington, DC 20555

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