

LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	N	Y	J	A	F	1	2	0	0	-	0	0	0	0	-	0	0	0	3	4	1	1	1	1	4		5															
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35															
LICENSEE CODE														LICENSE NUMBER										LICENSE TYPE										CAT 58									

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0	1	L	6	0	5	0	0	0	3	3	3	7	0	9	0	6	8	3	8	0	9	2	3	8	3	9																	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35															
REPORT SOURCE														DOCKET NUMBER										EVENT DATE										REPORT DATE									

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | While performing instrument surveillance procedure on "C" Average Power Range Monitor

0 3 | (APRM), the downscale trip level setting was found at 2.25%. This exceeded the

0 4 | Technical Specification Table 3.1-1 requirement of $\geq 2.5\%$. Redundant components

0 5 | were operable. There were no effects on public health or safety.

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0	9	I	A	11	E	12	E	13	I	N	S	T	R	U	14	X	15	Z	16						
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26						
SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE										COMP. SUBCODE		VALVE SUBCODE							
EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.																	
83		034		03		L		0																	
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER									
E18		Z19		Z20		Z21		0000		N23		Y24		N25		G080									
33		34		35		36		37		40		41		42		43		44		45		46		47	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The cause of "C" Average Power Range Monitor setpoint change was attributed to

1 1 | instrument drift. "C" Average Power Range Monitor was immediately adjusted to with-

1 2 | in the tolerance of Technical Specification 3.1-1 and placed an increased surveillance

1 3 | test frequency for trend observation. There were two operable channels remaining as

1 4 | specified in Technical Specification Table 3.1-1.

1	5	E	28	0	2	0	29	N/A	30	R	31	Surveillance Testing	32
7	8	9	10	11	12	13	14	15	16	17	18	19	20
FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION					
E28		020		N/A		R31		Surveillance Testing					
ACTIVITY		CONTENT		AMOUNT OF ACTIVITY		LOCATION OF RELEASE							
Z33		Z34		NA		NA							
7		8		9		10							
PERSONNEL EXPOSURES		NUMBER		TYPE		DESCRIPTION							
0		0		0		NA							
7		8		9		10							
PERSONNEL INJURIES		NUMBER		DESCRIPTION									
0		0		NA									
7		8		9		10							
LOSS OF OR DAMAGE TO FACILITY		TYPE		DESCRIPTION									
Z42		NA		NA									
7		8		9		10							
PUBLICITY		ISSUED		DESCRIPTION									
NA		NA		NA									
7		8		9		10							

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PDR ADOCK 05000333
S PDR

NRC USE ONLY

NAME OF PREPARER Douglas J. LindseyPHONE: (315) 342-3840 Ext. 300

James A. FitzPatrick
Nuclear Power Plant
P.O. Box 41
Litchfield, New York 13093
315 342.3840



Corbin A. McNeill, Jr.
Resident Manager

September 23, 1983
JAFP 83-0972

Dr. Thomas E. Murley, Regional Administrator
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA 19406

REFERENCE: DOCKET NO. 50-333 Licensee Event Report: 83-034/03L-0

Dear Mr. Murley:

We have enclosed the referenced Licensee Event Report in accordance with Section 6.0 of Technical Specifications and USNRC Regulatory Guide 1.16.

If there are any questions concerning this report, please contact Mr. Douglas J. Lindsey at (315) 342-3840, Extension 300.

Very truly yours,

A handwritten signature in cursive script, appearing to read 'Corbin A. McNeill, Jr.'.

CORBIN A. McNEILL, JR.
RESIDENT MANAGER

CAM:RTL:dmh
Enclosure

CC: USNRC Document Control Desk (1)
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Internal Power Authority Distribution
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LER/OR File

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