

TENNESSEE VALLEY AUTHORITY

DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT

MONTHLY OPERATING REPORT

TO THE

NUCLEAR REGULATORY COMMISSION

AUGUST 1, 1983 - AUGUST 31, 1983

UNIT 1

DOCKET NUMBER 50-327

LICENSE NUMBER DPR-77

UNIT 2

DOCKET NUMBER 50-328

LICENSE NUMBER DPR-79

Submitted By:

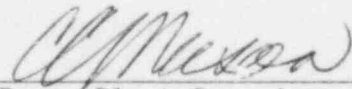

Power Plant Superintendent

TABLE OF CONTENTS

Operations Summary	1
Significant Operational Events	1-3
PORV's and Safety Valves Summary	4
Licensee Events and Special Reports	4-5
Offsite Dose Calculation Manual Changes	5
Operating Data	
Unit 1	6-8
Unit 2	9-11
Plant Maintenance Summary	12-16

Operations Summary

August, 1983

The following summary describes the significant operational activities for the month of August. In support of this summary, a chronological log of significant events is included in this report.

Unit 1

Unit 1 was critical for 673.8 hours, produced 755,890 MWH (gross), resulting in an average hourly gross load of 1,139,246 kW during the month. There are 87.44 full power days estimated remaining until the end of cycle 2 fuel. With a capacity factor of 85 percent, the target EOC exposure would be reached December 12, 1983. The capacity factor for the month was 85.9 percent.

There were not any reactor scrams, there were two manual shutdowns, and there was one power reduction during August.

Unit 2

The refueling/modification outage continues.

Significant Operational Events

Unit 1

<u>Date</u>	<u>Time</u>	<u>Event</u>
08/01/83	0001	Reactor in mode 1 at 100% power producing 1150 MWe.
08/23/83	2325	Began power reduction to 30% due to steam generator chemistry.
08/25/83	0550	Reactor at 30% power producing 272 MWe.
	1324	Generator taken off-line.
	1340	Reactor entered Mode 3.
	2005	Reactor entered Mode 4.
08/26/83	0445	Reactor entered Mode 3.

Significant Operational Events

Unit 1

(Continued)

<u>Date</u>	<u>Time</u>	<u>Event</u>
08/27/83	0040	Reactor taken critical
	0115	Reactor at 4% power and holding due to steam generator chemistry.
	0917	Reactor entered Mode 1.
	1026	The generator was tied on-line.
	1130	Reactor in Mode 1 at 30% power and holding due to steam generator chemistry.
08/28/83	0210	Began power ascension.
	0405	Reactor at 42% power and holding for heater drains clean-up.
	0620	Resumed power ascension.
	1848	Reactor at 85% power and producing 988MWe. "C" Condensate booster pump tripped on low oil pressure. Holding at 85% power.
	2300	Resumed power ascension.
08/29/83	0220	Reactor at 100% power producing 1146 MWe, #3 governor valve swinging \cong 10%.
08/31/83	0926	Reactor at 100%. Began unit shutdown due to problems with the steam generator chemistry. Investigation revealed that due to a series of events, the C.D.W.E. was dumped to the condensate system.
	1232	Unit was removed from the power grid.
	1430	Reactor entered Mode 3.
	2157	Reactor entered Mode 4.
	2359	Reactor in Mode 4 and cooling down to Mode 5 to clean-up the condensate and steam generators.

Significant Operational Events

(Continued)

Unit 2

Date

Time

Event

The refueling/modification outage continues.

Fuel Performance

Unit 1

The core average fuel exposure accumulated during August was 1021.13 MWD/MTU with the total accumulated core average fuel exposure of 7800.20 MWD/MTU.

Unit 2

The scheduled outage continues. Refueling process commenced on August 11 and continued for the remaining of the month. However, the reloading process had been temporarily suspended due to the clarity of the water. The refueling process constitutes a total core off-load (193 assemblies), a fuel insert shuffle and a total core reload; 68 of the 73 Region L assemblies were discharged and replaced with 68 Region P assemblies. The 5 remaining Region L assemblies will remain in the core for cycle 2.

Unit 2 (cycle 2) fuel bundle P-39 was returned to Westinghouse on 8/26/83 for repairs due to incorrect fuel design. It was discovered there was lower enrichment and the wrong size pellets in 19 fuel pins. It was received, satisfactorily inspected, and stored in the spent fuel pit on 8/29/83.

PORV'S and Safety Valves Summary

No PORV's or safety valves were challenged during the month.

Licensee Events and Special Reports

The following Licensee Event Reports (LER's) were sent during August 1983, to the Assistant Director of Nuclear Power (Operations) for reporting to the Nuclear Regulatory Commission.

Unit 1

<u>LER</u>	<u>SUBJECT</u>
SQRO-50-327/83096	Gaseous effluent radiation monitor 0-RM-90-101 for the auxiliary building exhaust stock was declared inoperable due to the loss of flow to the iodine sampler.
SQRO-50-327/83098	A glycol system containment isolation valve 1-FCV-61-191 was discovered failed. Investigation revealed a ruptured diaphragm in the valve actuator.
SQRO-50-327/83100	1-PCV-3-122 for train A AFW system was declared inoperable when it failed to open after a reactor trip. Investigation revealed the local manual control switch had been inadvertently placed in the closed position.
SQRO-50-327/83102	On July 22, 26, 30 and 31, 1983 a waste gas decay tank was found to have an oxygen concentration greater than 2%.
SQRO-50-327/83103	1-PCV-3-122 failed to open on demand during the performance of SI-276.
SQRO-50-327/83104	480v shutdown board 2B2-B was made inoperable during the performance of SMI-4A because the alternate supply was not energized when the board was transferred from normal to alternate supply.
SQRO-50-327/83106	1-FT-2-257 flow rate monitor for condenser vacuum exhaust was declared inoperable when it failed high. The transmitter drifted high due to condensation in the sense line.
SQRO-50-327/83108	On August 6, 8, and 9, 1983 the RCS subcooling margin monitor was declared inoperable due to the loss of the plant computer. Memory card A707 was found faulty due to shorted zener diode.

Licensee Events and Special Reports

Unit 2

<u>LER</u>	<u>SUBJECT</u>
SQRO-50-328/83097	Containment sump level channel 2-L-63-176 was declared inoperable due to reading high. The investigation indicated possible air bubbles in the reference leg.
SQRO-50-328/83099	During the performance of SI-111, five main steam line safety valves 2-1-512, -513, -517, -519, -521 were found with the lift setpoints out of tolerance.
SQRO-50-328/83101	Residual heat removal pump B began cavitating while the refueling cavity was being pumped down for maintenance on loop 4 RCS cold leg nozzle inspection plate.
SQRO-50-328/83105	Number 4 RCP underfrequency relay was found inoperable during the performance of SI-227. A defective control supply capacitor was found and the undervoltage timer failed.
SQRO-50-328/83107	During ice weighing per SI-106, Group 1, bays 1-8, row 1 average ice basket weight was found below the 1200 lb/basket minimum. Twenty additional baskets were found to weight below the 1200 lb minimum. The probable cause was due to sublimation.
SQRO-50-327/83109	It was discovered on August 16, 1983 that SI-270.2 did not meet the requirements of SR 4.8.3.1.A.3. SI-270.2 'Inspection of Molded Case and Lower Voltage Circuit Breaker Backup Fuses' was written to satisfy the temporary surveillance requirements that was intended for unit 1 only.

Special Reports

There were no special reports transmitted during the month of August.

Offsite Dose Calculation Manual Changes

There were no changes to the Sequoyah Nuclear Plant ODCM during the month.

OPERATING DATA REPORT

DOCKET NO. 50-327
DATE SEPTEMBER 13 1983
COMPLETED BY M. G. EDDINGS
TELEPHONE (615) 870-6543

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 1
2. REPORT PERIOD: AUGUST 1-31 1983
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBERS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: _____
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): _____
10. REASONS FOR RESTRICTIONS, IF ANY: _____

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.00	5831.00	19008.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	673.80	5203.21	12738.91
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	663.50	5102.70	12422.20
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	2170836.81	16654938.80	39937680.80
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	755890.00	5733520.00	13491056.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	728842.00	5527798.00	12963802.00
19. UNIT SERVICE FACTOR	89.18	87.51	65.35
20. UNIT AVAILABILITY FACTOR	89.18	87.51	65.35
21. UNIT CAPACITY FACTOR (USING MDC NET)	85.33	82.58	59.41
22. UNIT CAPACITY FACTOR (USING DER NET)	85.33	82.58	59.41
23. UNIT FORCED OUTAGE RATE	10.82	4.92	12.96
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): <u>Cycle 2 Refueling/Modification, December 2, 1983, 90 day Outage.</u>			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: <u>September 15, 1983</u>			

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-327
 UNIT NAME SEQUOYA ONE
 DATE SEPTEMBER 8, 1983
 COMPLETED BY M. Eddings
 TELEPHONE (615) 870-6196

REPORT MONTH AUGUST

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method Of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
21	830824	F	69.0	H	1				Manual Shutdown due to Steam Generator Water Chemistry above limits.
22	830831	F	11.5	H	1				Condensate Demineralized Waste Evaporator dumped to the condensate system. Steam generator water chemistry above limits.

1
 F: Forced
 S: Scheduled

2
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Cont. of Existing
 Outage
 5-Reduction
 9-Other

4
 Exhibit G-Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

5
 Exhibit I-Same Source

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-327
UNIT ONE
DATE SEPTEMBER 13, 1983
COMPLETED BY M. Eddings
TELEPHONE (615) 870-6543

MONTH AUGUST, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>1111</u>
2	<u>1111</u>
3	<u>1110</u>
4	<u>1109</u>
5	<u>1106</u>
6	<u>1106</u>
7	<u>1108</u>
8	<u>1098</u>
9	<u>1091</u>
10	<u>1109</u>
11	<u>1107</u>
12	<u>1112</u>
13	<u>1111</u>
14	<u>1114</u>
15	<u>1115</u>
16	<u>1113</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>1117</u>
18	<u>1116</u>
19	<u>1116</u>
20	<u>1113</u>
21	<u>1113</u>
22	<u>1115</u>
23	<u>1113</u>
24	<u>264</u>
25	<u>Off Line</u>
26	<u>Off Line</u>
27	<u>142</u>
28	<u>673</u>
29	<u>1098</u>
30	<u>1107</u>
31	<u>503</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

DOCKET NO. 50-328
DATE 9-13-83
COMPLETED BY D.C.DUPREE
TELEPHONE (615)870-6543

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 2
2. REPORT PERIOD: AUGUST 1-THRU-31, 1983
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1163.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1128.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBERS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: -----

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): -----

10. REASONS FOR RESTRICTIONS, IF ANY: -----

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.00	5831.00	10968.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.00	4750.77	8639.47
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	0.00	4715.95	8522.70
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.00	15401769.43	27556360.23
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	0.00	5299490.00	9381340.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0.00	5113057.00	9039347.60
19. UNIT SERVICE FACTOR	0.00	80.88	77.71
20. UNIT AVAILABILITY FACTOR	0.00	80.88	77.71
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.00	77.74	73.06
22. UNIT CAPACITY FACTOR (USING DER NET)	0.00	76.38	71.79
23. UNIT FORCED OUTAGE RATE	0.00	1.14	8.84
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			
October 7, 1983			

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-328
 UNIT NAME SEQUOYAH TWO
 DATE SEPTEMBER 13, 1983
 COMPLETED BY D. C. Dupree
 TELEPHONE (615) 870-6543

REPORT MONTH AUGUST

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method Of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
13	830719	S	744	C	4				Refueling outage continues.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Cont. of Existing
 Outage
 5-Reduction
 9-Other

⁴
 Exhibit G-Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (IER) File (NUREG-
 0161)

⁵
 Exhibit I-Same Source

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-328
 UNIT TWO
 DATE SEPTEMBER 13, 1983
 COMPLETED BY D. C. Dupree
 TELEPHONE (615) 870-6543

MONTH AUGUST, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>N/R</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

Plant Maintenance Summary

The following significant maintenance items were completed during the month of August 1983:

MECHANICAL MAINTENANCE

1. Replaced the oil pump on P.D. Pump 1C.
2. Replaced the internal element on 2AA Centrifugal Charging Pump.
3. Repaired the air-motor coupling, air-motor air leads and wafer valve in the transfer canal - unit 2.
4. Repacked the main steam check valves on unit 2.
5. Replaced the bonnet gasket in 2-FCV-74-1.
6. Replaced the yoke in 2-FCV-74-2.
7. Supported operations in handling of fuel - unit 2.
8. Completed modification to the upper seal housing and continuing RCP seal inspection.
9. Continuing thermal drilling in the ice condenser.
10. Started disassembly of internal element on 2BB centrifugal charging pump.

Electrical Maintenance

1. Continued the installation of the dimension 2000 phone system.
2. Completed walkdown inspection of the E-Field.
3. Completed reactor coolant pump modification on pumps 1 & 4.
4. Started modification on RCP #3.
5. Completed inspection of the motor leads of motors inside containment.
6. Continued 6900 and 480 volt motor insulation resistance testing.
7. Completed 69 unit 2 outage related maintenance request.
8. During a short unit 1 outage completed 8 unit 1 related maintenance request.
9. Completed an extensive check out of unit 2 annunciator system.

Plant Maintenance Summary

(Continued)

Instrument Maintenance

Unit 1

1. Rescaled PRT level transmitter 1-LT-68-312C for piping reconfiguration.
2. Completed monthly surveillance of UHI level switches. All switches were found within Tech Spec tolerance. This is the second consecutive month that all switches were found within tolerance.

Unit 2

1. Began scheduled calibrations and maintenance for the refueling outage which are 69% complete, completed calibration of EHC controller, and completed 83% of response time testing.
2. Completed modifications to Foxboro current repeaters and power supply modules.
3. Inspection and repair of MSR level controls 95% completed.
4. Installed a prototype Static O Ring level switch on the UHI water accumulator as a special test to determine accuracy of permanent level switches. The monthly surveillance of the level switches were completed. All switches were found within Tech Spec tolerance. This is the third consecutive month that all switches were found within tolerance.

Field Services Group

1. ECNs 2780/5200--Post-Accident Sampling Facility (Units 1 and 2)

Installation of the duct work for the HVAC systems serving the PASF continues. The HVAC fans have now been mounted on EL 706 in the Auxiliary building. The component cooling system piping tie-ins are complete. Work inside the polar crane wall of the unit 2 lower containment which routes the sampling tubing to their piping tie-in locations is complete. Work continues in the accumulator room to route the 8 sample tubes serving lower containment for unit 2. Installation of unit 2 containment conduit is in progress.

2. ECN 5429--Containment Hydrogen Mitigation System (Unit 2)

Lower containment - Conduit installation and cable pulling is complete in all areas of lower containment. Igniter installation and cable terminations are nearing completion on "A" train below elevation 731 in lower containment. Final Raychem of cable connections on "B" train is nearing completion inside lower containment below elevation 731.

Ice Condenser - Rigid conduit installation is complete in the ice condenser. The remaining conduit and junction box installation is nearing completion in the ice condenser.

Plant Maintenance Summary

(Continued)

Field Services (continued)

2. ECN 5429--Containment Hydrogen Mitigation System (Unit 2) (continued)

Upper Containment - Conduit installation in this area is complete. Additional work in this area is delayed until after refueling operations are complete.

Reactor Cavity - Work in this area is not expected to begin until after refueling operations are complete.

3. ENC 5009--ERCW Piping Changeout (Units 1 and 2)

Pipe change out of the ERCW header serving the RHR/CS/SIS pump room coolers 2A has been completed. Pipe fabrication is in progress for change out of ERCW pipe serving various pieces of equipment in the plant. Change out of these pipes is dependent upon the duration of equipment outage periods.

4. ECN 5645--Steam Generator Blowdown (SGBD) (Unit 2 Work)

All five outage condensate pipe tie-ins with isolation valves have been completed for the condensate piping serving the unit 2 SGBD heat exchangers. Remaining condensate piping and hanger installation will be completed after the unit 2 cycle 1 refueling outage.

SGBD piping serving the unit 2 SGBD heat exchangers has now been tied into the cooling tower blowdown piping. Work is in progress to tie-in the SGBD piping from the condensate demineralizer to the main condenser "A".

5. ECN 5596--Batch Neutralization System

The remaining mechanical and electrical work necessary for system operation was completed.

6. ECN 5198--Technical Support Center (TSC)

The TSC computer systems have been energized and are ready for the site acceptance test. Conduit is being installed inside the unit 2 containment during the present outage. Work is continuing in the TSC area on the fire detection system. Various cable pulls are continuing in the control building.

8. ECN 5642--N₂ Supply To The Condensate Storage Tanks (CST)

Work is complete inside CST "A" that installed the N₂ sparger and associated piping. Only painting remains inside the tank. Final pipe tie-in outside the tank remains. The tank vent piping has been installed as well as the tank pressure switch.

Plant Maintenance Summary

(Continued)

Field Services (continued)

9. ECN 2773,2775,2779,2923--Post Accident Radiation Monitoring (Unit 2)

The main control room panels 2-M-30 & 2-M-31 have been installed. Internal panel wiring is nearing completion. Field cable-connectors are being installed in preparation for cable terminations.

10. ECN 5440--6.9Kv Shutdown Board Degraded Voltage Circuitry

Internal panel wiring is complete on the various affected panels. Work has begun to terminate cables at the 6.9Kv shutdown boards.

11. ECN 5684--RCP Motor Oil Spray Shields

The No. 2 and No. 3 motor spray shield frames are complete. Sheetmetal was procured for the spraysields. Installation of the sheetmetal on the No. 2 spray shield is underway. Frame work on the No. 1 and No. 4 motors is delayed until these motors are replaced.

12. ECN 5847--Plant Fire Dampers

Seventy-two of the 89 plant fire dampers have been modified. Work also remains on this ECN to changeout some plant fire dampers.

13. ECN 5647--Unit 2 Main Feedpump Turbine Condenser (MFPT)
Condenser Air Removal Piping

This piping reroute is nearing completion. Hanger work and pipe tie-ins remain.

14. ECN 5608--Pressurizer Manway

Securing the pressurizer enclosure area which is to receive the manway insert is in work. The insert will be installed after refueling operations and head replacement is complete.

15. ECN 5856--Pressurizer Loop Seal Drain

Installation of this piping and the new pipe hangers is nearing completion. Modification of the existing pipe supports remain to be completed.

16. ECN 5773--Pressurizer Power Operated Relief Valve (PORV) and Piping
Changeout

All piping has been rerouted to receive the new PORVs. These valves are to be sent to the manufacturer for testing since their Watts Bar counterparts failed hot functional testing. Some hanger work continues.

Plant Maintenance Summary

(Continued)

Field Services (continued)

17. ECN 5743--Pressurizer Enclosure Platform

Work is nearing completion on this platform inside the top of the pressurizer enclosure. Only final installation of the grating and handrails remains.

Work is in progress to install the access platforms at the steam generator secondary side manway ports. Work is also in progress to install ladders, platforms and walkways to provide better access to the steam generators and the pressurizer.

18. ECN 2777--RPV Head Vent

The lower containment piping installation is approximately 90 percent complete. Some pipe hanger installation is continuing in lower containment. All piping and hangers to be installed in the reactor cavity are prefabricated but cannot be installed until refueling operations and RPV head replacement is complete.

Due to some late design changes, the electrical portion of this modification is behind the original schedule. Relocation of the control valve flow modulators from the reactor building to the auxiliary building is the major cause of the additional work. Work remains to install conduit and cable for the flow modulators; repulling of some cable inside containment serving the system valves; installing the remaining instruments and handswitches in the main control room panel Z-M-4; and making all cable terminations.

19. ECN 2776--Containment Pressure Monitoring

Mounting of the main control room panel 2-M-9 pressure recorders was completed this month. Internal panel wiring is scheduled to begin next month. Cable terminations remain as well.

20. ECN 5644--Condensate Hotwell makeup/Dumpback

The portion of this piping that ties in to the condensate storage tank "A" is now complete. No further work will be conducted during the present unit 2 outage.

21. ECN 5769--Feedwater Chemical Feed Tie-in Relocation

Work has begun to remove the existing tubing and to reroute the tubing to the tie-in locations on the auxiliary feedwater piping.

TENNESSEE VALLEY AUTHORITY
Sequoyah Nuclear Plant
P. O. Box 2000
Soddy-Daisy, Tennessee 37379

September 15, 1983

Nuclear Regulatory Commission
Office of Management Information
and Program Control
Washington, DC 20555

Gentlemen:

Enclosed is the August 1983 Monthly Operating Report to the NRC for Sequoyah Nuclear Plant.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

C. C. Mason
C. C. Mason

Power Plant Superintendent

Enclosure

cc (Enclosure):

Director, Region II
Director, Office of Management
Nuclear Regulatory Commission
101 Marietta Street
Suite 3100
Atlanta, GA 30303 (1 copy)

Director, Office of Inspection
and Enforcement
Nuclear Regulatory Commission
Washington, DC 20555 (10 copies)

Mr. A. Rubio, Director
Mr. Bill Lavalley, NSAC
Electric Power Research Institute
P. O. Box 10412
Palo Alto, CA 94304 (1 copy)

Mr. R. C. Goodspeed
MNC 461
Westinghouse Electric Corporation
P. O. Box 355
Pittsburgh, PA 15230 (1 copy)

Director, Office of Management
Information and Program Control
Nuclear Regulatory Commission
Washington, DC 20555 (2 copies)

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339 (1 copy)

IE24
11