



Northern States Power Company

Monticello Nuclear Generating Plant  
2807 West Hwy 75  
Monticello, Minnesota 55362-9637

November 15, 1994

Report Required by  
10 CFR Part 50, Section 50.73

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT  
Docket No. 50-263 License No. DPR-22

LER 94-016  
False High Reactor Pressure During Testing Causes  
a Partial Containment Isolation

The Licensee Event Report for this occurrence is attached. This report contains the following new NRC commitment:

The Excess Flow Check Valve Test Procedure will be revised to ensure the Shutdown Cooling isolation valves are closed during testing that could cause automatic actuation.

Please contact Tom Parker at (612) 295-1014 if you require further information.

*Roger O Anderson*

Roger O Anderson  
Director  
Licensing and Management Issues

c: Regional Administrator - III NRC  
Sr Resident Inspector, NRC  
NRR Project Manager, NRC  
State of Minnesota,  
Attn: Kris Sanda

Attachment

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PDR ADDCK 05000263  
S PDR

*JE 22/1*

NRC FORM 366 (5-92)		U.S. NUCLEAR REGULATORY COMMISSION			APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95						
<b>LICENSEE EVENT REPORT (LER)</b>  <small>(See reverse for required number of digits/characters for each block)</small>								ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.			
FACILITY NAME (1) <b>MONTICELLO NUCLEAR GENERATING PLANT</b>						DOCKET NUMBER (2) <b>05000 - 263</b>		PAGE (3) <b>1 OF 3</b>			
TITLE (4) <b>False High Reactor Pressure During Testing Causes a Partial Containment Isolation</b>											
EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER	
10	17	94	94	016	00	11	15	94	FACILITY NAME	DOCKET NUMBER	
										05000	
										05000	
OPERATING MODE (9)		N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
				20.402(b)		20.405(c)		X 50.73(a)(2)(iv)		73.71(b)	
POWER LEVEL (10)		0%		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)	
				20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER	
				20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		(Specify in Abstract	
				20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		below and in Text, NRC	
				20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		Form 388A)	
LICENSEE CONTACT FOR THIS LER (12)											
NAME Tom Parker						TELEPHONE NUMBER (Include Area Code) 612-295-1014					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	
SUPPLEMENTAL REPORT EXPECTED (14)									EXPECTED SUBMISSION DATE (15)		
YES (IF YES, COMPLETE EXPECTED SUBMISSION DATE)					X NO						

ABSTRACT (LIMIT TO 1400 SPACES, I.E., APPROXIMATELY 15 SINGLE-SPACED TYPEWRITTEN LINES) (16)  
NCR FORM 366 (5-91)

While flushing instrument lines with demineralized water, a partial containment isolation occurred during a refueling outage. The pressure in one of the instrument lines increased to the setpoint for the high pressure isolation of the Shutdown Cooling valves. The test procedure will be modified to close these valves prior to performance of the test.

NRC FORM 366A (5-92)		U.S. NUCLEAR REGULATORY COMMISSION		APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95	
<b>LICENSEE EVENT REPORT (LER)</b> <b>TEXT CONTINUATION</b>				ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.	
FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)	
MONTICELLO NUCLEAR GENERATING PLANT		05000 263		YEAR 94	SEQUENTIAL NUMBER 016
				REVISION NUMBER 00	PAGE (3) 2 of 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

### Description

On October 17, 1994 during a refueling outage, a partial containment isolation (EIS System Code: JM) occurred during performance of the Excess Flow Check Valve Test Procedure, 0255-20-ID-1. During this test, demineralized water (at approximately 130 psig) is injected into instrument lines for the purpose of flushing them. When one of the two instrument lines, that sense pressure for the high pressure isolation of the Shutdown Cooling system, was flushed, the Shutdown Cooling valves isolated. The setpoint for this isolation is 75 psig (actual setpoint, due to head corrections, is around 50 to 60 psig). The logic for this non-Engineered Safety Feature is one out-of-two. The logic closes both Shutdown Cooling Isolation valves (MO-2029 and MO-2030)(EIS System Code: KE), which are containment isolation valves.

### Cause

This procedure is performed immediately following the ASME Section XI Leakage Test of the Reactor Recirculation System (EIS System Code: AD). The Shutdown Cooling valves are closed during this leakage test. This outage it was necessary to place Shutdown Cooling in service between the leakage test and the back flushing of the instrument lines. Following cooling the reactor water temperature to 120 degrees F, the pumps used for Shutdown Cooling were secured and the isolation valves were left in the open position. Although the Excess Flow Check Valve Test procedure required the pumps used for Shutdown cooling to be placed in "pull-to-lock", no requirement existed to re-close the Shutdown Cooling valves. The cause of this event is lack of the proper procedural prerequisite in the Excess Flow Check Valve Test Procedure.

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MONTICELLO NUCLEAR GENERATING PLANT		05000 263		YEAR 94	SEQUENTIAL NUMBER 016
				REVISION NUMBER 00	PAGE (3) 3 of 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

### Analysis

This event is reportable per 10 CFR Part 50, Section 50.73(a)(2)(iv) since an automatic actuation of a Engineered Safety Feature occurred. The signal was an invalid high pressure signal and was not an Engineered Safety Feature signal. However, the signal closed Engineered Safety Feature valves (primary containment isolation valves) that were not part of the exempted systems:

Reactor water clean-up system;  
 Control room emergency ventilation system;  
 Reactor building ventilation system;  
 Fuel building ventilation system;  
 or  
 Auxiliary building ventilation system,

and therefore this event is reportable.

Shutdown cooling was not in use at this time; the Reactor Water Cleanup system, in the heat reject mode, was removing decay heat. Therefore, this event did not cause any additional increase in reactor coolant temperature. There were no consequences associated with this event. A temperature increase could have occurred if shutdown cooling had been removing heat from the core (however, adequate time would have been available to return shutdown cooling to service prior to reaching any safety concern).

### Corrective Actions

The Excess Flow Check Valve Test Procedure will be revised to ensure the Shutdown Cooling isolation valves are closed during testing that could cause automatic actuation.

### Failed Component Identification

None

### Previous Similar Events

None

TRANSMITTAL MANIFEST  
NORTHERN STATES POWER COMPANY  
NUCLEAR LICENSING DEPARTMENT  
MONTICELLO NUCLEAR GENERATING PLANT

False High Reactor Pressure During Testing  
Causes a Partial Containment Isolation

Manifest Date: November 16, 1994

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Monticello Internal Site Distribution:

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NL Monti Plant File

Kaleen Hilsinhoff.....USAR File.....Yes\_\_\_ No\_X\_  
Steve Ludders.....NRC Commitment.....Yes\_X\_ No\_\_\_  
Lila Imholt.....Monti OC Sec.....Yes\_\_\_ No\_X\_ - 10, No dist to OC members above  
Mel Opstad.....Monti SAC Sec.....Yes\_X\_ No\_\_\_ - 6  
\*Mail Room.....Monti Posting.....Yes\_\_\_ No\_X\_ - 7

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\* Advance Distribution made by Site Licensing  
Tom Parker to make OC Distribution  
Correspondence Date : November 15, 1994



## NRC Correspondence Manifest Instructions

Document: LER 94-016

Manifest: M-5 (Use M-1 w/ additions indicated on M-5)

Advance Dist.

Additional Dist:

Delete Dist:

Dave Bollig Addressee's identified  
as Oc members

### Special Manifest Requirments

USAR Yes \_\_\_\_\_ No X (Send copy to Kaleen Hilsenhoff)

NRC Commitment Yes X No        (Send copy to Steve Ludders)

Resp Reminder Yes ☐ No ☒

Resp Start Date	Resp Period	Resp Due Date
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Lic Contact: \_\_\_\_\_

Resp Start Date \_\_\_\_\_ Resp Period \_\_\_\_\_ Resp Due Date \_\_\_\_\_

Action: \_\_\_\_\_

Lic Contact: \_\_\_\_\_

Resp Start Date	Resp Period	Res'n Due Date
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Action:

Lic Contact:

Resp Start Date \_\_\_\_\_ Resp Period \_\_\_\_\_ Resp Due Date \_\_\_\_\_

Action:

**Lic Contact:**

Monti OC Sec Yes \_\_\_\_\_ No X (10 copies to L Imholt, withhold distribution to OC ID addressees)

Monti SAC Sec Yes X No        (1 copy to SAC Sec File, <sup>5</sup> Copies to non-NSP SAC members in weekly mailing)

Monti Posting Yes \_\_\_\_\_ No X (7 advance dist copies to J McNabb)

Ext comm Dept Yes X No     

\* Add Comment to Transmittal Manifest  $\rightarrow$

NSP H:\WORD\NRCCORR\MAN-FORM.DOC "Tom Parker to make OC Distribution"