

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
 UNIT Fort Calhoun Station
 DATE October 13, 1983
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 536-4733

MONTH September, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>440.1</u>
2	<u>440.4</u>
3	<u>440.9</u>
4	<u>441.9</u>
5	<u>443.3</u>
6	<u>444.4</u>
7	<u>445.2</u>
8	<u>445.5</u>
9	<u>445.5</u>
10	<u>444.5</u>
11	<u>446.1</u>
12	<u>448.1</u>
13	<u>450.1</u>
14	<u>450.9</u>
15	<u>453.1</u>
16	<u>453.7</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>453.4</u>
18	<u>452.9</u>
19	<u>452.1</u>
20	<u>454.1</u>
21	<u>455.6</u>
22	<u>457.4</u>
23	<u>457.9</u>
24	<u>457.8</u>
25	<u>457.5</u>
26	<u>456.1</u>
27	<u>454.9</u>
28	<u>454.4</u>
29	<u>453.4</u>
30	<u>452.7</u>
31	<u> </u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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 PDR ADOCK 05000285
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OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE October 13, 1983
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: September, 1983
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 461
7. Maximum Dependable Capacity (Net MWe): 438
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	6,552.0	87,793.0
12. Number Of Hours Reactor Was Critical	720.0	4,294.9	67,684.9
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	720.0	4,196.0	67,193.5
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,073,897.6	5,997,955.6	83,514,504.0
17. Gross Electrical Energy Generated (MWH)	340,574.0	1,852,050.0	27,275,529.7
18. Net Electrical Energy Generated (MWH)	324,093.5	1,757,502.0	26,087,536.4
19. Unit Service Factor	100.0	64.0	76.5
20. Unit Availability Factor	100.0	64.0	76.5
21. Unit Capacity Factor (Using MDC Net)	102.8	61.2	64.5
22. Unit Capacity Factor (Using DER Net)	94.2	56.1	62.5
23. Unit Forced Outage Rate	0.0	1.2	3.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

1984 refueling outage scheduled to start around March 16, 1984 and
will last approximately two months.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH September, 1983

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun Station
 DATE October 13, 1983
 COMPLETED BY J. P. Matthews
 TELEPHONE (402) 536-4733

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns for the month of September, 1983.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending September 1983 .

1. Scheduled date for next refueling shutdown. March 1984
2. Scheduled date for restart following refueling. May 1984
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes
 - a. If answer is yes, what, in general, will these be?

A Technical Specification Change

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. _____
 - c. If no such review has taken place, when is it scheduled? _____
4. Scheduled date(s) for submitting proposed licensing action and support information. Tech. Specs. - February 1984
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
6. The number of fuel assemblies:

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>265</u>	"
c) spent fuel pool storage capacity	<u>483</u>	"
d) planned spent fuel pool storage capacity	<u>728</u>	"
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1985

Prepared by

J K E usper

Date October 1, 1983

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

September, 1983
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at 100% power the entire month of September. The new spent fuel racks began arriving on site September 19, 1983. Three operators and one engineer are participating in hot license training at Combustion Engineering's simulator in Windsor, Connecticut.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
83-007	During routine power operations at approximately 100% power, the RPS "B" channel Axial Power Distribution (APD) positive trip setpoint was noticed to be operating out of specification tolerance in the non-conservative direction. The "B" channel of the RPS for Axial Power Distribution was immediately placed in bypass. During the event the redundant Axial Power Distribution channels of the RPS (i.e., A, C, & D) were verified to be operating satisfactorily and as designed.

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance test which resulted in an Operation Incident and is not reported elsewhere in the report:

<u>Operation Incident</u>	<u>Deficiency</u>
OI-1758 ST-ISI-CVCS-3	The 96-hour engineering review signoff was not completed within the required 96-hours.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description/Summary</u>
SP-FE-10	<p>Inspection of Spent Fuel Using CE Curb Mount Equipment.</p> <p>This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved inspection of spent fuel assemblies in the spent fuel pool. Only external measurements were taken and all movement of fuel assemblies was done in accordance with established procedures.</p>
SP-UF6-1	<p>Uranium Hexafluoride Storage Cylinder External Visual Inspection.</p> <p>This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the inspection of storage cylinders.</p>
SP-FAUD-1	<p>Fuel Assembly Uplift Condition Detection.</p> <p>This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.</p>

System Acceptance Committee Packages for September, 1983:

<u>Package</u>	<u>Description/Analysis</u>
DCR 77-91	<p>Backflush Connections on Raw Water Pumps</p> <p>This modification added a connection to "C" raw water pump to allow for backflushing before startup of the pump which improves the reliability of the pump. This modification has no adverse effect on the safety analysis.</p>
EEAR FC-78-36	<p>Repair of AS-594</p> <p>This modification changed out an auxiliary steam valve (non-safety related). This modification has no adverse effect on the safety analysis.</p>
EEAR FC-78-48	<p>Steam Generator Blowdown Isolation Valves HCV-137AB and HCV-1388AB</p> <p>This modification changed the internal trim and body gaskets of these valves which improved their reliability. This modification has no adverse effect on the safety analysis.</p>

<u>Package</u>	<u>Description/Analysis</u>
EEAR FC-78-70	<p>Auto Close Breaker Mode Switch Wiring Change</p> <p>This modification changed a set of contacts on one of the diesel auto close breaker mode switches. The operation of the switch was not effected. This modification has no adverse effect on the safety analysis.</p>
EEAR FC-79-64	<p>Install Unions on AC Safety/Relief Valves</p> <p>This modification added unions to component cooling relief valves. The operation of the valves and the setpoints of the valves remained unchanged. This modification has no adverse effect on the safety analysis.</p>
EEAR FC-82-139	<p>TSC Batteries</p> <p>This modification changed out the TSC batteries with those of a higher quality. This modification has no adverse effect on the safety analysis.</p>
EEAR FC-83-60 (SRDCO 83-33)	<p>Moisture Separator Bypass on Auxiliary Building Crane</p> <p>This modification provided a bypass for an air line moisture separator. This modification has no adverse effect on the safety analysis.</p>
DCR 77-10	<p>Dual Setpoint Operation of PORV's</p> <p>This modification provided dual setpoint operation for the PORV's. One setpoint for low temperature, and one for operating temperature. This modification has no adverse effect on the safety analysis.</p>
EEAR FC-79-200	<p>Stack Flow Monitor Improvement</p> <p>This modification improved the readability and operation of the stack flow monitor. This modification has no adverse effect on the safety analysis.</p>
EEAR MR-FC-81-128, Part 1 Only	<p>Waste Filters, Part 1 (Pilot Filters Installation)</p> <p>This modification installed two new tie-in points to the Liquid Radioactive Waste System which allow the addition of a temporary Pilot Filter to the waste system. These tie-ins were designed to meet piping class 152. This modification has no adverse effect on the safety analysis.</p>

<u>Package</u>	<u>Description/Analysis</u>
EEAR FC-80-123	Raw Water Pump Room Stairs This modification replaced the old raw water pump room stairway with one that was less steep. This modification has no adverse effect on the safety analysis.
EEAR FC-80-102	Sixth Stage Extraction Erosion This modification replaced elbows on the sixth stage extraction piping with a more suitable material than carbon steel for resisting erosion. This modification has no adverse effect on the safety analysis.
EEAR FC-79-201	Storeroom Lighting This modification added more lighting to CQE storage areas in the storeroom. This modification has no adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

There are no new leak rate results to report.

F. CHANGES IN PLANT OPERATING STAFF

None

G. TRAINING

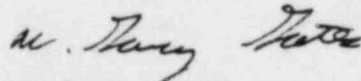
General employee training was conducted as scheduled. Operator requalification training included physics review, properties of fluids, hydrostatics, thermal dynamics, fire brigade, heat balance, heat transfer, hydrodynamics and emergency procedures. Non-licensed operator training was included. Emergency plan training was conducted for plant personnel. Maintenance personnel received training on plant systems. Three operators and one engineer participated in hot license training at Combustion Engineering's simulator in Windsor, Connecticut.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No. 75	Allows an increase in the storage capacity for the spent fuel pool from 483 whole fuel assemblies to 729 whole fuel assemblies.
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II. MAINTENANCE (Significant Safety Related)

None



W. Gary Gates
Manager
Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102
402/536-4000

October 13, 1983
LIC-83-266

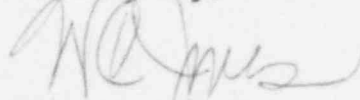
Mr. Richard C. DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Reference: Docket No. 50-285

Dear Mr. DeYoung:

Please find enclosed ten (10) copies of the September
Monthly Operating Report for the Fort Calhoun Station Unit
No. 1.

Sincerely,



W. C. Jones
Division Manager
Production Operations

WCJ/TPM:jmm

Enclosures

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
NRC File

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