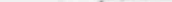


## LICENSEE EVENT REPORT

CONTROL BLOCK: 

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	T	N	S	N	P	1	7	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5	
7	8	LICENSEE CODE						14	LICENSE NUMBER												25	LICENSE TYPE						30	57 CAT 5b	

CON'T

REPORT SOURCE: L 6 0 5 0 0 0 3 2 7 7 0 9 1 0 8 3 8 1 0 0 7 8 3 9

60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

DICTIONARY NUMBER: 60-69

EVENT DATE: 70-79

REPORT DATE: 80-89

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 Unit 1 in mode 4. At 0949 CDT on 09/10/83, the containment internal pressure

03 | limit of +.3 psig relative to the annulus pressure was exceeded. This event

0 4 required entry into LCO 3.6.1.4. There was no effect upon public health or

05 | safety. Previous occurrences - two (SQRO-50-328/83017 and 83033).

06

0	7
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0	8
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SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP SUBCODE	VALVE SUBCODE
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0 9      S A (11)    x (12)    z (13)    z z z z z z (14)    z (15)    z (16)

7	8	9	10	11	12	13	14	15	16	17	18	19	20		
SEQUENTIAL						OCCURRENCE						REPORT		REVISION	

LER/RO REPORT	EVENT YEAR	REPORT NO.	ZONE	REPORT TYPE	REPORT NO.
(17)	8 2	1 1 0	1	1	1

NUMBER 

21	22
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24	25	26
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32
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ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	(22)	ATTACHMENT SUBMITTED	NPDD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
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X (18) Z (19) Z (20) 0 0 0 0 Y (23) N (24) Z (25) Z 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (21)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0	Internal pressure increased due to normal air leakage into containment without
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venting. Factors contributing to the event were failure of the gamma spectrometry

1 system preventing sample analysis prior to venting and poor communication between

1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80	81	82	83	84	85	86	87	88	89	90	91	92	93	94	95	96	97	98	99	100
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1 3 Chemistry Lab and Operations personnel. The containment pressure was returned

1-4 1 within the limit at 1355 CDT.

7. 8. 9

STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
1.0	1.0	1.0	1.0	1.0

1	5	G	28	0	0	0	19	NA	A	31	Operator observation
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ACTIVITY	CONTENT	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
RELEASED	OF RELEASE	(35)	(36)

1	6	Z	33	Z	34	NA	NA
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8	9	10	11	44	45
PERSONNEL EXPOSURES					

NUMBER			TYPE		DESCRIPTION
1	7	1000	(37)	Z	(38) NA

PERSONNEL INDIVIDUEL

NUMBER	DESCRIPTION
41	

8 0 0 0 (40) NA

LOSS OF OR DAMAGE TO FACILITY		(43)
TYPE	DESCRIPTION	

9	Z	(42)	NA
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10  
PUBLICITY (46) 8310170597 831007 NRC USE ONLY

ISSUED		DESCRIPTION	
2	0	N	NA

PDR ADUCK 05000327  
S PDR

19 68 69

Sequoyah Nuclear Plant

LER SUPPLEMENTAL INFORMATION

SQRO-50-327/83119

Technical Specification Involved: 3.6.1.4

Reported Under Technical Specification: 6.9.1.13.b

Date of Occurrence: 09/10/83

Time of Occurrence: 0949 CDT

Identification and Description of Occurrence:

At 0949 CDT on 09/10/83, the containment internal pressure limit of +.3 psig relative to the annulus pressure was exceeded. This event required entry into LCO 3.6.1.4.

Conditions Prior to Occurrence:

Unit 1 in mode 4 with RCS temperature and pressure at 260 degrees F and 340 psig.

Apparent Cause of Occurrence:

Internal pressure increased due to normal air leakage into containment without venting.

Analysis of Occurrence:

Poor communication between Chemistry Lab and Operations personnel prevented the SI-410 ventilation package (required for proper release authorization) from being performed until pressure was near the limit. Failure of the gamma spectrometry system prevented the containment atmosphere sample analysis prior to the pressure exceeding the limit.

Corrective Action:

The sample was analyzed, containment vented, and pressure returned within the limit at 1355 CDT on 09/10/83.

The importance of improved communications has been discussed with Chemistry Lab and Operations personnel.

Failure Data:

Reference: SQRO-50-328/83017 and 83033.

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

83 OCT 11 P 1:51

October 7, 1983

Mr. James P. O'Reilly, Director  
U.S. Nuclear Regulatory Commission  
Suite 2900  
101 Marietta Street, NW  
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET  
NO. 50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE  
REPORT SORC-50-327/83119

The enclosed report provides details concerning the exceeding of the  
containment internal pressure limit of +.3 psig relative to the annulus.  
This report is submitted in accordance with Sequoyah unit 1 Technical  
Specification 6.9.1.13.b.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

*J. A. Green*  
J. A. Green

Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Records Center  
Institute of Nuclear Power Operations  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, Georgia 30339

NRC Inspector, Sequoyah

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