

LICENSEE EVENT REPORT

CONTROL BLOCK:

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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	T	N	S	N	P	1	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5	
7	8	9	LICENSEE CODE					14	15	LICENSE NUMBER										25	26	LICENSE TYPE					30	ST CAT		36

CON'T

REPORT SOURCE L 6 0 5 0 0 0 3 2 7 7 0 9 2 6 8 3 8 1 0 0 7 8 3 9

60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 With unit 1 in mode 1 (100% Rx power) at 0825 on 09/26/83, it was discovered that

03 the monthly power range incore-excore channel comparison was not performed in the

04 specified time interval. Upon even discovery, all four channels were declared

05 | inoperable and the unit complied with LCO 3.0.3. There was no effect upon

06 public health and safety. Previous occurrences - none.

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0	8	
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SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE
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0	9	I	A	11	A	12	X	13	Z	Z	Z	Z	Z	Z	14	Z	15	Z	16
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7 8 9 10 11 12 12 18 19 20

SEQUENTIAL OCCURRENCE REPORT REVISION

LER/RO REPORT	EVENT YEAR	REPORT NO.	SEQUENCE CODE	REPORT TYPE	REVISION NO.
(17)					

NUMBER

8	3
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 21 22

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 23

1	1	7
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 24 25 26

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 27

0	1
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 28 29

1

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0

 32

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
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X	18	X	19	Z	20	Z	21	0	0	0	0	Y	23	N	24	Z	25	Z	9	9
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CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The unit had been in mode 5, cold shutdown, prior to returning to full power on

11 09/18/83. Using a very conservative interpretation of the technical specifications

1 2 | the surveillance should have been performed on 09/19/83, but was not completed until

1322 CDT on 09/26/83. Failure to perform the surveillance is considered a personnel

1.4 | error, and those involved have been instructed concerning technical specification

requirements.

FACTORY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION

1	5	E	28	1	0	0	19	NA	B	31	Surveillance tests
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ACTIVITY	CONTENT	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
RELEASED	OF RELEASE	(35)	(36)

1	6	Z	33	Z	3A	NA	NA
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PERSONNEL EXPOSURES

NUMBER				TYPE	DESCRIPTION
1	2	0	0	37	7 38 NA

PERSONNEL INJURIES

NUMBER	DESCRIPTION
41	

8	0	0	0	(40)	NA
2		11	12		

LOSS OF OR DAMAGE TO FACILITY		(43)
TYPE	DESCRIPTION	

19 2 42 NA

PUBLICITY (AC) 8310170592 831007 NRC USE ONLY

ISSUED	DESCRIPTION	PDR ADCK 05000327
2 0	N (44) NA	S PDR

8 9 10 68 69

Sequoyah Nuclear Plant

LER SUPPLEMENTAL INFORMATION

830-50-327/83117

Technical Specification Involved: 3.3.1.1

Reported Under Technical Specification: 6.9.1.12.b

Date of Occurrence: 09/26/83

Time of Occurrence: 0825 CDT

Identification and Description of Occurrence:

During a review of surveillance requirements by the Reactor Engineering Section, it was discovered that SI-79, Monthly Power Range Incore-Excore Channel Calibration, was not performed in the specified time interval. Upon event discovery, all four power range channels were declared inoperable, and the unit complied with LCO 3.0.3.

Conditions Prior to Occurrence:

Unit 1 in mode 1 (100% Rx power).

Unit 2 in mode 5 (0% Rx power).

Apparent Cause of Occurrence:

The failure to perform the surveillance in the specified time interval was caused by personnel error. Persons involved were using effective full power days instead of calendar days to track the due date for the next performance.

Analysis of Occurrence:

SI-79 was performed on 08/11/83 and the unit was in mode 5 cold shutdown from approximately 09/01/83 until 09/12/83. The Reactor Engineering Section performs SI-126, Hot Channel Factor Determination, Units 1 and 2, and SI-79 concurrently, and it was incorrectly assumed that both surveillances had the same surveillance interval (31 EFPD) since they are core physics dependent. Since the unit had recently been in cold shutdown and core burnup was less than 31 effective full power days, it was not realized that the surveillance was due.

Using 31 calendar days as the frequency interval for the incore-excore axial flux difference is a very conservative interpretation of the technical specifications. However, until a technical specification has been approved, the plant will continue to use this interpretation for performing SI-79.

Corrective Action:

Upon event discovery, a performance of SI-79 was initiated and satisfactorily completed at 1322 CDT on 09/26/83. Personnel involved were instructed on using 31 calendar days as a bases for tracking SI-79. A technical specification revision will be submitted to change the incore-excore calibration times based on effective full power days.

Failure Data:

None.

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

83 OCT 11 P 1:51

October 7, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET
NO. 50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE
REPORT SPRO-50-327/83117

The enclosed report provides details concerning the failure to perform
surveillance within the specified time limits on reactor trip power range
channels. This report is submitted in accordance with Sequoyah unit 1
Technical Specification 6.9.1.12.b.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

J. A. Loggery
J. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Sequoyah

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