Commonwealth Edison
LaSalle County Nuclear Station
2601 N. 21st. Rd.
Marseilles, Illinois 61341
Telephone 815/357-6761

November 11, 1994

U.S. Nuclear Regulatory Commission
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Washington, D.C. 20555

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for October, 1994.

D. J. Ray
Station Manager
LaSalle County Station

DJR/tmb

Enclosure

cc: John B. Martin, Regional Administrator - Region III
NRC Senior Resident Inspector - LaSalle
IL Department of Nuclear Safety - LaSalle
IL Department of Nuclear Safety - Springfield, IL
NRR Project Manager - Washington, D.C.
GE Representative - LaSalle
Regulatory Assurance Supervisor - LaSalle
Licensing Operations Director - Downers Grove
Nuclear Fuel Services Manager - General Office
Off-Site Safety Review Senior Participant - Downers Grove
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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

October 1994

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

TABLE OF CONTENTS
(UNIT 1)

I. INTRODUCTION (UNIT 1)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
1	0000	Reactor critical, Generator on-line at 1135 Mwe.
8	0300	Reduced power level to 1050 Mwe due to system load.
	0900	Increased power level to 1135 Mwe.
15	0100	Reduced power level to 900 Mwe to perform testing on the Motor Driven Reactor Feed Pump and control card changeout on the 'A' Turbine Driven Reactor Feed Pump.
	0400	Increased power level to 1135 Mwe.
28	0400	Reduced power level to 830 Mwe to perform scram time testing.
	1000	Increased power level to 1140 Mwe.
30	1500	Reduced power level to 1050 Mwe to perform a rod set.
	1600	Increased power level to 1140 Mwe.
31	2400	Reactor critical, Generator on-line at 1140 Mwe.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION
(None)

C. LICENSEE EVENT REPORTS (Unit 1)
(None)

D. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 1)

2. Average Daily Unit Power Level (See Table 2)

3. Unit Shutdowns and Significant Power Reductions (See Table 3)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 1)

1. Safety Relief Valve Operations
(None)

2. Major Changes to Radioactive Waste Treatment Systems
(None)

3. Static O-Ring Failures
(None)

4. Changes to the Off-Site Dose Calculation Manual
(None)

TABLE 1
D.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE November 10, 1994
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: October 1994 — GROSS HOURS IN REPORTING PERIOD: 745
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 — MAX DEPEND CAPACITY (MWe-Net): 1,036; DESIGN ELECTRICAL RATING (MWe-N 1,078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):
4. REASONS FOR RESTRICTION (IF ANY):

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	745.0	4,022.7	65,349.6
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	745.0	3,839.1	63,852.8
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWHt)	2,454,696	11,381,568	188,707,205
10. ELECTRICAL ENERGY GENERATED (MWHe-Gross)	835,880	3,799,813	63,054,184
11. ELECTRICAL ENERGY GENERATED (MWHe-Net)	809,102	3,627,132	60,475,886
12. REACTOR SERVICE FACTOR (%)	100.0	55.1	68.8
13. REACTOR AVAILABILITY FACTOR (%)	100.0	55.1	70.5
14. UNIT SERVICE FACTOR (%)	100.0	52.6	67.2
15. UNIT AVAILIBILITY FACTOR (%)	100.0	52.6	67.2
16. UNIT CAPACITY FACTOR (USING MDC) (%)	104.8	48.0	61.5
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	100.7	46.1	59.1
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	17.3	8.2

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 2
D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE November 10, 1994
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: October 1994

DAY	POWER	DAY	POWER
1	1,090	17	1,090
2	1,092	18	1,093
3	1,091	19	1,093
4	1,090	20	1,094
5	1,089	21	1,092
6	1,089	22	1,093
7	1,089	23	1,094
8	1,083	24	1,091
9	1,088	25	1,095
10	1,086	26	1,095
11	1,089	27	1,084
12	1,091	28	1,021
13	1,091	29	1,088
14	1,079	30	1,131
15	1,039	31	1,096
16	1,089		

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER # if applicable)
(None)						

SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system load maintenance and surveillance activities.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

October 1994

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

TABLE OF CONTENTS
(UNIT 2)

I. INTRODUCTION (UNIT 2)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

Day	Time	Event
1	0000	Reactor critical, Generator on-line at 840 Mwe due to Condensate pump limitations.
3	1600	Reduced power level to 640 Mwe due to a '25A' Feedwater Heater isolation.
	2100	Increased power level to 830 Mwe.
6	0000	Reduced power level to 735 Mwe to place the '25A' Feedwater Heater on-line.
	0200	Increased power level to 830 Mwe.
	0900	Reduced power level to 810 Mwe to perform scram time testing.
	0930	Increased power level to 835 Mwe.
7	0100	Reduced power level to 810 Mwe to perform scram time testing.
	0130	Increased power level to 840 Mwe.
9	0030	Reduced power level to 800 Mwe to perform scram time testing.
	1800	Increased power level to 835 Mwe.
11	0300	Reduced power level to 740 Mwe to perform scram time testing.
	0700	Increased power level to 830 Mwe.
	1300	Reduced power level to 730 Mwe to perform scram time testing.
	1830	Increased power level to 820 Mwe.
12	0300	Reduced power level to 730 Mwe to perform scram time testing.
	0600	Increased power level to 835 Mwe.
	1400	Reduced power level to 715 Mwe to perform scram time testing.
	1700	Increased power level to 840 Mwe.
13	0200	Reduced power level to 740 Mwe to perform scram time testing.
	0800	Increased power level to 830 Mwe.
	1530	Reduced power level to 725 Mwe to perform scram time testing.
	1800	Increased power level to 825 Mwe.
14	0230	Reduced power level to 680 Mwe to perform scram time testing.

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)
(continued)

Day	Time	Event
14	0600	Increased power level to 835 Mwe.
	1300	Reduced power level to 700 Mwe to place the 'A' Condensate/Condensate Booster pump on-line.
	2200	Increased power level to 1100 Mwe.
16	0300	Reduced power level to 845 Mwe to perform scram time testing.
	1300	Increased power level to 1135 Mwe.
17	0300	Reduced power level to 980 Mwe to perform replacement of scram pilot solenoid valves.
	0500	Increased power level to 1060 Mwe.
	1930	Reduced power level to 850 Mwe to perform scram time testing.
18	0100	Increased power level to 1100 Mwe.
	2200	Reduced power level to 845 Mwe to perform scram time testing.
19	0300	Increased power level to 1125 Mwe.
	1057	Reactor scram due to Electro Hydraulic Control system oscillations.
24	0126	Reactor critical.
	2015	Generator on-line at 60 Mwe.
25	1900	Power level at 1065 Mwe.
26	0800	Reduced power level to 740 Mwe due to loss of the 'C' Condensate/Condensate Booster pump.
	1600	Increased power level to 810 Mwe.
27	0230	Reduced power level to 740 Mwe to perform replacement of a scram solenoid pilot valves.
	0300	Increased power level to 800 Mwe.
28	0300	Reduced power level to 730 Mwe to perform replacement of a scram solenoid pilot valves.
	0400	Increased power level to 800 Mwe.
	2200	Reduced power level to 700 Mwe to place the 'C' Condensate/Condensate Booster pump on-line.
29	1900	Increased power level to 1035 Mwe.
30	1300	Reduced power level to 850 Mwe to perform a rod set.
	2000	Increased power level to 1075 Mwe.
31	2400	Reactor critical, Generator on-line at 1090 Mwe.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION
(None)

C. LICENSEE EVENT REPORTS (Unit 2)

LER No.	Date	Description
94-007	10/04/94	Division I group 2 and group 4 isolation due to a blown fuse.
94-008	10/19/94	Reactor scram due to Electro Hydraulic Control system oscillations.

D. DATA TABULATIONS (Unit 2)

1. Operating Data Report (See Table 1)

2. Average Daily Unit Power Level (See Table 2)

3. Unit Shutdowns and Significant Power Reductions (See Table 3)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 2)

1. Safety Relief Valve Operations

Valve	Date of Actuation	Type of Actuation	Description
2B21-F013K	10/19/94	Automatic	Reactor Scram (LER# 94-008)
2B21-F013F	10/19/94	Automatic	Reactor Scram (LER# 94-008)
2B21-F013G	10/19/94	Automatic	Reactor Scram (LER# 94-008)
2B21-F013S	10/19/94	Automatic	Reactor Scram (LER# 94-008)
2B21-F013U	10/19/94	Automatic	Reactor Scram (LER# 94-008)
2B21-F013D	10/19/94	Automatic	Reactor Scram (LER# 94-008)

2. Major Changes to Radioactive Waste Treatment Systems
(None)

3. Static O-Ring Failures
(None)

4. Changes to the Off-Site Dose Calculation Manual
(None)

TABLE 1
D.1 OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE November 10, 1994
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: October 1994 — GROSS HOURS IN REPORTING PERIOD: 745
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 — MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):
4. REASONS FOR RESTRICTION (IF ANY):

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	634.5	6,818.4	63,744.0
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	615.7	6,639.2	62,521.5
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWHt)	1,610,525	20,914,004	189,014,457
10. ELECTRICAL ENERGY GENERATED (MWHe-Gross)	543,564	7,083,332	63,067,972
11. ELECTRICAL ENERGY GENERATED (MWHe-Net)	523,268	6,856,924	60,612,815
12. REACTOR SERVICE FACTOR (%)	85.2	93.5	72.4
13. REACTOR AVAILABILITY FACTOR (%)	85.2	93.5	74.4
14. UNIT SERVICE FACTOR (%)	82.6	91.0	71.1
15. UNIT AVAILIBILITY FACTOR (%)	82.6	91.0	71.1
16. UNIT CAPACITY FACTOR (USING MDC) (%)	67.8	90.7	66.5
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	65.2	87.2	63.9
18. UNIT FORCED OUTAGE FACTOR (%)	17.4	5.3	10.8

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 2
D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE November 10, 1994
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: October 1994

DAY	POWER	DAY	POWER
1	801	17	1,013
2	801	18	1,053
3	793	19	472
4	804	20	-12
5	801	21	-12
6	796	22	-12
7	806	23	-12
8	806	24	9
9	803	25	712
10	805	26	819
11	785	27	769
12	792	28	768
13	791	29	863
14	824	30	1,023
15	1,065	31	1,025
16	1,049		

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 2)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER # if applicable)
6	941019	F	129.3	A	3	Reactor scram due to Electro Hydraulic Control system oscillations (LER# 94-008)

SUMMARY OF OPERATION:

The unit remained on-line throughout most of the month. Several minor power reductions were required due to maintenance and surveillance activities. The unit experienced a reactor scram on 10/19/94 due to oscillations in the Electro Hydraulic Control system. The unit was returned to service on 10/24/94.