

## LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6 7 8 9 10										(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)									
0 1 2 3 4 5 6 7 8 9										0 H D B S 1 2 3 4 5 6 7 8 9 0 1 2 3 4 5 6 7 8 9									
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CON'T										REPORT SOURCE									
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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)																			
0 2										(NP-33-83-56) During the performance of ST 5061.02, the Containment Local Leak Rate									
0 3										Test (LLRT), four containment isolation valves CCl407B, SA2010, DR2012A, and CCl411B									
0 4										were found with leakage rates in excess of T.S. 3.6.1.2b and c. There was no danger									
0 5										to the health and safety of the public or station personnel. Two containment isolation									
0 6										valves are always provided in series. The backup isolation valves for all four valves									
0 7										showed essentially zero leakage.									
0 8																			
0 9																			
SYSTEM CODE										CAUSE CODE									
S D 11										E 12									
CAUSE SUBCODE										COMP. SUBCODE									
X 13										V A L V E X 14									
VALVE SUBCODE										REPORT TYPE									
X 15										D 16									
LER/RO REPORT NUMBER										EVENT YEAR									
17										8 3									
ACTION TAKEN										FUTURE ACTION									
B 18										Z 19									
EFFECT ON PLANT										SHUTDOWN METHOD									
Z 20										Z 21									
HOURS										ATTACHMENT SUBMITTED									
0 0 0 0 22										Y 23									
NPRD-4 FORM SUB.										PRIME COMP. SUPPLIER									
Y 24										Z 25									
COMPONENT MANUFACTURER										REVISION NO.									
Z 9 9 9 26										0 27									
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)																			
1 0										The cause of CCl407B, DR2012A, and SA2010 failing their leak rate test was component									
1 1										failures. DR2012A had rust and dirt on the seat. CCl407B had a torn liner, and									
1 2										SA2010 had dirty and worn internals. They have been repaired and retested satisfac-									
1 3										torily. CCl411B failed its leak rate test due to procedure deficiency. The valve									
1 4										operator's torque limit switch was out of adjustment due to an inadequate procedure.									
FACILITY STATUS										METHOD OF DISCOVERY									
D 28										B 31									
% POWER										DISCOVERY DESCRIPTION (32)									
0 0 0 29										During performance of ST 5061.02									
ACTIVITY CONTENT										LOCATION OF RELEASE (36)									
RELEASED OF RELEASE																			
Z 33										NA									
PERSONNEL EXPOSURES																			
NUMBER TYPE DESCRIPTION (39)																			
0 0 0 37										Z 38									
PERSONNEL INJURIES																			
NUMBER DESCRIPTION (41)																			
0 0 0 40										NA									
LOSS OF OR DAMAGE TO FACILITY																			
TYPE DESCRIPTION (43)																			
Z 42										NA									
PUBLICITY																			
ISSUED DESCRIPTION (45)																			
N 44										NA									
8310120239 830921																			
PDR ADOCK 05000346																			
S																			
PDR																			
NRC USE ONLY																			
68 69																			
419-259-5000, Ext. 254/535																			
DVR 83-106																			
NAME OF PREPARER																			
Don Missig/Ted Lang																			
PHONE:																			

TOLEDO EDISON COMPANY  
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE  
SUPPLEMENTAL INFORMATION FOR LER NP-33-83-56

DATE OF EVENT: August 22, 1983

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Leakage of Containment Isolation Valves  
Found by Local Leak Rate Testing

Conditions Prior to Occurrence: The unit was in Modes 5 and 6, with Power (Mwt) = 0 and Load (Gross MWe) = 0.

Description of Occurrence: During the performance of ST 5061.02, The Containment Local Leak Rate Test (LLRT), four containment isolation valves, CC1407B, SA2010, DR2012A, and CC1411B, were found with leakage rates in excess of Technical Specifications 3.6.1.2 (b and c).

Technical Specification 3.6.1.2.b limits combined leakage from type b and c tests to less than 0.6La, approximately 600,000 standard cubic centimeters of air per minute (SCCM), and Technical Specification 3.6.1.2.c limits combined emergency ventilation "bypass leakage" to less than 0.015La, approximately 15,000 SCCM. Out of the above listed valves, only valves SA2010 and DR2012A fall under jurisdiction of Technical Specification 3.6.1.2.c. The action statement of Technical Specification 3.6.1.2 which requires restoration of leakage rates to within limits prior to increasing the Reactor Coolant System temperature above 200°F, has been met.

Designation of Apparent Cause of Occurrence: The cause of valves CC1407B, DR2012A, and SA2010 failing their leak rate test was component failure. Valve DR2012A had rust and dirt on the seat. Valve CC1407B had a torn liner. Valve SA2010 had dirty and worn internals.

The cause of CC1411B failing its leak rate test was due to procedure deficiency. Valve CC1411B had a limitorque operator with the torque limit switch out of adjustment, which caused the valve not to close to its seated position. This was caused by the adjustment procedure in MP 1410.32, Removal and Repair of Limitorque Valve Controls, being inadequate.

Analysis of Occurrence: There was no danger to the health and safety of the public or station personnel. Two containment isolation valves are always provided in series. The backup (or second) isolation valve for CC1407B, SA2010, DR2012A, and CC1411B all showed essentially zero leakage.

Corrective Action: Each of the valves listed above as being defective due to component failure have been repaired and retested satisfactorily. Procedure modification T-7430 was written for MP 1410.32 to improve the limitorque adjustment procedure. CC1411B was readjusted under procedure modification T-7430 and when it was retested, it had zero leakage. All other limitorque motor actuated valves were also retested if the limitorque actuator was worked on since the last local leak rate test was performed.

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DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE  
SUPPLEMENTAL INFORMATION FOR LER NP-33-83-56  
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SA2010 has been found to leak during previous leak rate tests and, therefore, the valve upstream will be isolated as well as SA2010 being maintained closed. The line, station air to containment, will also be blown down, and the valve's position, mentioned above, will be checked monthly.

Failure Data: Similar occurrences were reported in Licensee Event Reports NP-33-80-52 (80-042) and NP-33-82-31 (82-027).

LER #83-044

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<u>VALVE</u>	<u>PENETRATION</u>	<u>VALVE DESCRIPTION</u>	<u>TECHNICAL SPECIFICATION*</u>	<u>LEAKAGE IN SCCM</u>	<u>CORRECTIVE MWO</u>	<u>RETEST DATE</u>	<u>RETEST LEAKAGE IN SCCM</u>
CC1407B	4	Component cooling water return header containment outer isolation motor actuated butterfly valve	3.6.1.2.b	610,000	1-83-4392-00	9/3/83	2899
SA2010	42A	Station air to containment isolation valve	3.6.1.2.c	68,600	1-83-4600-00	9/6/83	736
DR2012A	13	Containment Sump Pump 1-1 discharge line - containment isolation valve - Control Room operated	3.6.1.2.c	350,000 (0 after stroking valve)	1-83-4629-00	9/15/83	0
CC1411B	3	Component cooling water supply header containment outer isolation motor actuated butterfly valve	3.6.1.2.b	Valve did not close to seated position	1-83-4693-01 1-83-4305-02	9/6/83	0

\* Technical Specification 3.6.1.2.b limit equals approximately 600,000 SCCM  
Technical Specification 3.6.1.2.c limit equals approximately 15,000 SCCM

ATTACHMENT 1

NOTICE

ATTACHED IS A RE-ISSUE OF LOG NO. K83-1320, LICENSEE EVENT REPORT NO. 83-044. THE PREVIOUS COPY HAD THE INCORRECT LER NUMBER, 83-043, ON THE TRANSMITTAL LETTER AND ON THE LICENSEE EVENT REPORT FORM.

PLEASE DESTROY ALL PREVIOUS COPIES OF THIS REPORT AND REPLACE WITH THE ATTACHED CORRECTED COPY.



September 21, 1983

Log No. K83-1320  
File: RR2 (NP-33-83-56)

Docket No. 50-346  
License No. NPF-3

Mr. James G. Keppler  
Regional Administrator, Region III  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

LER No. 83-044  
Davis-Besse Nuclear Power Station Unit 1  
Date of Occurrence: August 22, 1983

Enclosed are three copies of Licensee Event Report 83-043 which are being submitted in accordance with Technical Specification 6.9 to provide 30 day written notification of the subject occurrence.

Yours truly,

Terry D. Murray  
Station Superintendent  
Davis-Besse Nuclear Power Station

TDM/ljk

Enclosures

cc: Mr. Richard DeYoung, Director  
Office of Inspection and Enforcement  
Encl: 30 copies

Mr. Norman Haller, Director  
Office of Management and Program Analysis  
Encl: 3 copies

Mr. Walt Rogers  
NRC Resident Inspector  
Encl: 1 copy

SEP 30 1983

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