

Docket No. 50-336
B15013

Attachment 1

Millstone Nuclear Power Station, Unit No. 2
Proposed Revision to Technical Specifications

Containment Leakage Type A Test Schedule
Marked-up Pages

October 1994

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CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of $\leq L_a$, 0.50 percent by weight of the containment air per 24 hours at P_a , 54 psig.
- b. A combined leakage rate of $\leq 0.60 L_a$ for all penetrations and valves subject to Type B and C tests when pressurized to P_a .
- c. A combined leakage rate of $\leq 0.017 L_a$ for all penetrations identified in Table 3.6-1 as secondary containment bypass leakage paths when pressurized to P_a .

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75 L_a$, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60 L_a$, or (c) with the combined bypass leakage rate exceeding $0.017 L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50.

- a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals* during shutdown at P_a (54 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.

*The test interval for conducting a Type A test shall be extended to allow the second Type A test, within the second ten-year service period, to be conducted during Cycle 11 refueling outage. This extension expires upon completion of Cycle 11 refueling outage.

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Three Type A tests (Overall Integrated Containment Leakage Tests) shall be conducted at approximately equal intervals during shutdown at a pressure not less than P_0 , 54 psig, during each 10-year service period.*

- * The third Type A test for the second 10-year period shall be conducted during the thirteenth refueling outage. As a result, the duration of the second 10-year service period will be extended to the end of the thirteenth refueling outage.

3/4.6 CONTAINMENT SYSTEMSBASES3/4.6.1 PRIMARY CONTAINMENT3/4.6.1.1 CONTAINMENT INTEGRITY

Primary CONTAINMENT INTEGRITY ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the accident analyses. This restriction, in conjunction with the leakage rate limitation, will limit the site boundary radiation doses to within the limits of 10 CFR 100 during accident conditions.

3/4.6.1.2 CONTAINMENT LEAKAGE

The limitations on containment leakage rates ensure that the total containment leakage volume will not exceed the value assumed in the accident analyses at the peak accident pressure of 54 psig, P_a. As an added conservatism, the measured overall integrated leakage rate is further limited to $\leq 0.75 \text{ L}^3$ during performance of the periodic tests to account for possible degradation of the containment leakage barriers between leakage tests.

The surveillance testing for measuring leakage rates are consistent with the requirements of Appendix "J" of 10 CFR 50, with the option of the use of the mass point method for performing leakage calculations.

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3/4.6.1.3 CONTAINMENT AIR LOCKS

The limitations on closure and leak rate for the containment air locks are required to meet the restrictions on CONTAINMENT INTEGRITY and leak rate given in Specifications 3.6.1.1 and 3.6.1.2. The limitations on the air locks allow entry and exit into and out of the containment during operation and ensure through the surveillance testing that air lock leakage will not become excessive through continuous usage.

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An exemption has been granted from the requirements of 10CFR50, APPENDIX J, SECTION III.D.1.(a). The exemption removes the requirement that the third Type A test for each 10-year period be conducted when the plant is shutdown for the 10-year plant inservice inspections (Reference license Amendment No. _____).

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Attachment 2

Millstone Nuclear Power Station, Unit No. 2
Proposed Revision to Technical Specifications

Containment Leakage Type A Test Schedule
Retyped Pages

October 1994

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of $\leq L_a$, 0.50 percent by weight of the containment air per 24 hours at P_a , 54 psig.
- b. A combined leakage rate of $\leq 0.60 L_a$ for all penetrations and valves subject to Type B and C tests when pressurized to P_a .
- c. A combined leakage rate of $\leq 0.017 L_a$ for all penetrations identified in Table 3.6-1 as secondary containment bypass leakage paths when pressurized to P_a .

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75 L_a$, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60 L_a$, or (c) with the combined bypass leakage rate exceeding $0.017 L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50.

- a. Three Type A tests (Overall Integrated Containment Leakage Tests) shall be conducted at approximately equal intervals during shutdown at a pressure not less than P_a , 54 psig, during each 10-year service period.*

*The third Type A test for the second 10-year period shall be conducted during the thirteenth refueling outage. As a result, the duration of the second 10-year service period will be extended to the end of the thirteenth refueling outage.

3/4.6 CONTAINMENT SYSTEMS

BASES

3/4.6.1 PRIMARY CONTAINMENT

3/4.6.1.1 CONTAINMENT INTEGRITY

Primary CONTAINMENT INTEGRITY ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the accident analyses. This restriction, in conjunction with the leakage rate limitation, will limit the site boundary radiation doses to within the limits of 10 CFR 100 during accident conditions.

3/4.6.1.2 CONTAINMENT LEAKAGE

The limitations on containment leakage rates ensure that the total containment leakage volume will not exceed the value assumed in the accident analyses at the peak accident pressure of 54 psig, P_a . As an added conservatism, the measured overall integrated leakage rate is further limited to $\leq 0.75 L_a$ during performance of the periodic tests to account for possible degradation of the containment leakage barriers between leakage tests.

The surveillance testing for measuring leakage rates are consistent with the requirements of Appendix "J" of 10 CFR 50, with the option of the use of the mass point method for performing leakage calculations.

An exemption has been granted from the requirements of 10CFR50, Appendix J, Section III.D.1.(a). The exemption removes the requirement that the third Type A test for each 10-year period be conducted when the plant is shutdown for the 10-year plant inservice inspections (Reference License Amendment No. _____).

3/4.6.1.3 CONTAINMENT AIR LOCKS

The limitations on closure and leak rate for the containment air locks are required to meet the restrictions on CONTAINMENT INTEGRITY and leak rate given in Specifications 3.6.1.1 and 3.6.1.2. The limitations on the air locks allow entry and exit into and out of the containment during operation and ensure through the surveillance testing that air lock leakage will not become excessive through continuous usage.

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Attachment 3

Millstone Nuclear Power Station, Unit No. 2
Proposed Revision to Technical Specifications

Containment Leakage Type A Test Schedule
Timeline for the Second 10-Year Service Period

October 1994

TIMELINE FOR THE SECOND 10-YEAR SERVICE PERIOD

Without Approval of Technical Specification Revision and Grant of Appendix J Exemption Request

