



Northern States Power Company

Prairie Island Nuclear Generating Plant

1717 Wakonade Dr. East  
Welch, Minnesota 55089

January 18, 1995

10 CFR Part 50  
Section 50.73

U S Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT  
Docket Nos. 50-282 License Nos. DPR-42  
50-306 DPR-60

Discovery That One Snubber Is Not  
Being Tested As Required Due To Personnel Error

The Licensee Event Report for this occurrence is attached. In the report, we have made one new NRC commitment:

Prior to the next Unit 2 refueling, the snubber will be added to the surveillance procedure that governs the refueling-interval visual inspections.

Please contact us if you require additional information related to this event.

*Michael D. Wadley for*

Roger O Anderson  
Director  
Licensing and Management Issues

c: Regional Administrator - Region III, NRC  
NRR Project Manager, NRC  
Senior Resident Inspector, NRC  
Kris Sanda, State of Minnesota

Attachment

9501250370 950118  
PDR ADDCK 05000306  
S PDR

*IF22  
11*

## LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

## FACILITY NAME (1)

Prairie Island Nuclear Generating Plant U2

## DOCKET NUMBER (2)

05000 306

## PAGE (3)

1 OF 3

TITLE (4) Discovery That One Cooling Water System Snubber Is Not Being Tested As Required

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
12	20	94	94	-- 04 --	00	01	18	95	Prairie Island U1	05000 282
									FACILITY NAME	DOCKET NUMBER
										05000
OPERATING MODE (9)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
			20.402(b)			20.405(c)			50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		100	20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)	73.71(c)
			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)	OTHER
			20.405(a)(1)(iii)		X	50.73(a)(2)(i)			50.73(a)(2)(viii)(A)	(Specify in Abstract below and in Text, NRC Form 366A)
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)	
			20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)	

## LICENSEE CONTACT FOR THIS LER (12)

NAME

Arne A Hunstad

TELEPHONE NUMBER (Include Area Code)

612-388-1121

## COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

## SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE).	X	NO
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EXPECTED  
SUBMISSION  
DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On December 20, 1994, both units were at 100% power. A 10-year InService Inspection of pipe hangers had been performed. The inspection identified a hydraulic snubber on cooling water piping to No. 23 Fan-Coil Unit, in location 2-CWH-621, which appeared to have a slight fluid leak. The snubber was declared inoperable, removed for testing, and a replacement snubber was installed; those activities used about 6 hours of the 72-hour Limiting Condition for Operation of Technical Specification 3.12. The removed snubber passed its functional test. Though the snubber appeared to be leaking a small amount of fluid, the fluid level in the reservoir was adequate. While researching the maintenance history of the snubber, it was determined that the snubber had been installed during a modification in 1983, and that the visual inspections required by Technical Specification 4.13 have not been performed. The visual inspections are scheduled for each refueling, though the required inspection interval is dependent upon prior inspection results and can be as long as 48 months. The previous visual inspection of this snubber was done in 1984 during the previous 10-year InService Inspection.

NRC FORM 366A (5-92)		U.S. NUCLEAR REGULATORY COMMISSION		APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95	
LICENSEE EVENT REPORT (LER) TEXT CONTINUATION				ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.	
FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)	
Prairie Island Unit 2		05000 306		YEAR 94	SEQUENTIAL NUMBER -- 04 --
				REVISION NUMBER 00	PAGE (3) 2 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

### EVENT DESCRIPTION

On December 20, 1994, both units were at 100% power. A 10-year InService Inspection of pipe hangers had been performed. The inspection identified a hydraulic snubber (EIIS Component Identifier: SNB) on cooling water piping to No. 23 Fan-Coil Unit, in location 2-CWH-621, which appeared to have a slight fluid leak. The snubber was declared inoperable, removed for testing, and a replacement snubber was installed; those activities used about 6 hours of the 72-hour Limiting Condition for Operation of Technical Specification 3.12. The removed snubber passed its functional test. Though the snubber appeared to be leaking a small amount of fluid, the fluid level in the reservoir was adequate. While researching the maintenance history of the snubber, it was determined that the snubber had been installed during a modification in 1983, and that the visual inspections required by Technical Specification 4.13 have not been performed. The visual inspections are scheduled for each refueling, though the required inspection interval is dependent upon prior inspection results and can be as long as 48 months. The previous visual inspection of this snubber was done in 1984 during the previous 10-year InService Inspection.

### CAUSE OF THE EVENT

Cause of the event is apparent oversight by the project engineer at the time of snubber installation during system modification. Information from the modification package was incorporated into the 10-year InService Inspection program, but not into the surveillance program required by Technical Specification 4.13.

### ANALYSIS OF THE EVENT

The event is reportable pursuant to 10CFR50.73(1)(2)(i)(B) since visual inspection of the snubber required by Technical Specification 4.13 was not performed. Inspection and testing of the snubber shows that it was operable at all times during the period when required visual inspections were not performed. Health and safety of the public were unaffected.

### CORRECTIVE ACTION

When the visual inspection showed evidence of fluid leakage, the snubber was declared inoperable, removed for further testing, and a replacement snubber was installed.

The removed snubber was functionally tested satisfactorily. Though the snubber appeared to be leaking a small amount of fluid, the fluid level in the reservoir was adequate. The inspection and testing done showed the

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snubber to be operable, but it was determined that visual examinations had not been done as required.

Prior to the next Unit 2 refueling, the snubber will be added to the surveillance procedure that governs the refueling-interval visual inspections.

The event was discussed with involved personnel.

This event will be reviewed by the Modification Process Re-Engineering Team, which is charged with improving the modification process.

#### FAILED COMPONENT IDENTIFICATION

None.

#### PREVIOUS SIMILAR EVENTS

Previous similar events involving test program deficiencies have been reported but none of those involved snubbers.