

NRC FORM 366 (12-81)		U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT		APPROVED BY OMB 3150-0011 EXPIRES 4-30-82	
CONTROL BLOCK: 1		(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)			
<div style="display: flex; justify-content: space-between;"> 0 1 M D C C N 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5 </div>					
7 8 9		14 15		23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40	
7 8		60 61		68 69 74 75 80	
CON'T					
<div style="display: flex; justify-content: space-between;"> 0 1 R L 6 0 5 0 0 0 3 1 7 7 0 8 0 1 8 0 8 0 8 1 9 8 3 9 </div>					
7 8		60 61		68 69 74 75 80	
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10					
0 2		During normal operation, at 0230 on 7/29/80, #12 Control Room Air Condi-			
0 3		tioning (CRAC) compressor tripped and would not restart. The redundant			
0 4		AC unit remained operable during the event as required by T.S. 3.7.6.1.			
0 5		#12 CRAC unit was returned to service at 1430 on 7/29/80. At 1230 on			
0 6		8/1/80 #12 CRAC compressor tripped on overcurrent. The redundant AC unit			
0 7		remained operable during the event as required by T.S. 3.7.6.1. #12 CRAC			
0 8		unit was returned at service at 1400 on 8/1/80. Similar event: 79-19 (U1)			
7 8		90			
0 9		<div style="display: flex; justify-content: space-between;"> S G B C X X X X X X Z Z </div>			
7 8		9 10		20 21	
17		EVENT YEAR		REVISION NO.	
<div style="display: flex; justify-content: space-between;"> 8 0 0 3 8 0 3 X 1 </div>		21 22		32 33	
ACTION TAKEN		EFFECT ON PLANT		SHUTDOWN METHOD	
<div style="display: flex; justify-content: space-between;"> E Z Z Z 0 0 0 0 N Y A B 3 5 0 </div>		33 34 35 36		37 38 39 40 41 42 43 44 45 46 47	
7 8		9 10		20 21	
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27					
1 0		Test was performed and compressor Amps were found to be higher than			
1 1		specifications found on nameplate data. Load demand was adjusted by			
1 2		regulating the unloader valve until Amps met nameplate data. To in-			
1 3		crease AC reliability, the unipressure valves have been replaced with			
1 4		hot-gas bypass and pressure regulating valves.			
7 8		90			
1 5		<div style="display: flex; justify-content: space-between;"> E 1 0 0 N/A A Operator Observation </div>			
7 8		9 10		20 21	
1 6		<div style="display: flex; justify-content: space-between;"> Z Z N/A N/A </div>			
7 8		9 10		20 21	
1 7		<div style="display: flex; justify-content: space-between;"> 0 0 0 Z N/A </div>			
7 8		9 10		20 21	
1 8		<div style="display: flex; justify-content: space-between;"> 0 0 0 N/A </div>			
7 8		9 10		20 21	
1 9		<div style="display: flex; justify-content: space-between;"> Z N/A </div>			
7 8		9 10		20 21	
2 0		<div style="display: flex; justify-content: space-between;"> N N/A </div>			
7 8		9 10		20 21	
NAME OF PREPARER		J.S. Lagiewski/R.W. L'Heureux		PHONE: 301-269-4492/4503	

BALTIMORE GAS AND ELECTRIC COMPANY

P.O. BOX 1475

BALTIMORE, MARYLAND 21203

NUCLEAR POWER DEPARTMENT
CALVERT CLIFFS NUCLEAR POWER PLANT
LUSBY, MARYLAND 20657

August 19, 1983

Dr. Thomas E. Murley
Regional Administrator
U. S. Nuclear Regulatory Commission
Region 1
631 Park Avenue
King of Prussia, PA 19406

Docket No. 50-317
License No. DPR 53

Dear Dr. Murley:

Attached is LER 80-38/3X, Revision 1, as required per Technical Specification 6.9.

Should you have any questions regarding this report, we would be pleased to discuss them with you.

Very truly yours,

LBR Russell

L. B. Russell
Plant Superintendent

LBR:RWL:bsb

cc: Director, Office of Management Information
and Program Control
Messrs: A. E. Lundvall, Jr.
J. A. Tiernan

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11