

LICENSEE EVENT REPORT

U.S. NUCLEAR REGULATORY COMMISSION

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 1 L Q A D 2 2 0 0 0 - 0 0 0 - 0 0 0 3 4 1 1 1 1 4 5
7 8 9 14 15 25 26 30 37 CAT 38

CON'T
01 REPORT SOURCE L 6 0 5 0 0 0 2 6 5 7 0 18 1 6 3 3 8 0 9 0 8 8 3 9
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 At 1515 hours, on August 16, 1983, following recalibration of the Suppression Pool
03 water temperature indication, QIS 52-1, the temperature was found to be 96 degrees
04 F. This is contrary to the maximum temperature limitation specified in Technical
05 Specification 3.7.A.1.c.1. A Reactor shutdown was immediately initiated. Adequate
06 cooling was still available to provide complete condensation of steam which would
07 be produced during a design basis accident; therefore, the safety consequences of
08 this event are minimal.
7 8 9 80

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
S A 11 X 12 X 13 I N S T R U 14 E 15 Z 16
7 8 9 10 11 12 13 18 19 20
17 LER/RO REPORT NUMBER EVENT YEAR OCCURRENCE CODE REPORT TYPE REVISION NO.
8 3 0 1 2 0 3 L 0
21 22 23 24 26 27 28 29 30 31 32
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
E 18 Z 19 B 20 Z 21 0 0 0 1 N 23 N 24 X 25 L 0 0 7 26
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The cause of this event was instrument calibration drift. The temperature indicator
11 was indicating a lower temperature than the actual water temperature. Suppression
12 Pool cooling was initiated immediately; and the temperature was within Technical
13 Specification limits by 1550 hours.
14
7 8 9 80

15 FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION
E 28 0 5 8 29 NA B 31 Routine Quarterly Calibration
7 8 9 10 11 12 13 44 45 46 80
16 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE
Z 33 Z 34 NA NA
7 8 9 10 11 44 45 80

17 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION
0 0 0 37 Z 38 NA
7 8 9 10 11 12 13 80

18 PERSONNEL INJURIES NUMBER DESCRIPTION
0 0 0 40 NA
7 8 9 10 11 12 80

19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION
Z 42 NA
7 8 9 10 80

20 PUBLICITY ISSUED DESCRIPTION
N 44 NA
7 8 9 10 80

NAME OF PREPARER P Cochran

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Commonwealth Edison

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NJK-83-315

September 8, 1983

J. Keppler, Regional Administrator
Office of Inspection and Enforcement
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

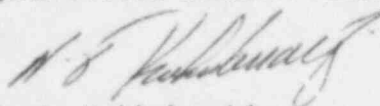
Reference: Quad-Cities Nuclear Power Station
Docket Number 50-265, DPR-30, Unit Two
Appendix A, Section 3.7.A.1.c

Enclosed please find Reportable Occurrence Report Number RO 83-12/03L-0
for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of
Technical Specification 6.6.B.2.b; operation in a degraded mode permitted
by a limiting condition of operation.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION


N. J. Kalivianakis
Station Superintendent

NJK:DGC/bb

Enclosure

cc B. Rybak
A. Morrongiello
INPO Records Center

SEP 14 1983

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