

# LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01
NCBEP2200-0000-0034111145

CON'T  
01
 REPORT SOURCE L6050-0324708158380914839

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

02 While performing the loop calibration of the remote shutdown panel reactor vessel  
03 level monitoring instrumentation, PT-55.3PC, as reactor vessel level transmitter  
04 2-B21-LT-N026A was being returned to service, both Unit No. 2 reactor recirculation  
05 pumps tripped due to actuation of ATWS/RPT instruments 2-B21-LT-N024A-2 and N024B-2.  
06 At this time of this event, the unit was at power operation. This event did not affect  
07 the health and safety of the public.

08 Technical Specifications 3.3.5.2, 3.3.6.1, 6.9.1.9.b

SYSTEM CODE IB11
 CAUSE CODE D12
 CAUSE SUBCODE Z13
 COMPONENT CODE INSTRU14
 COMP. SUBCODE T15
 VALVE SUBCODE Z16

LER/RO REPORT NUMBER 17
 EVENT YEAR 83
 SEQUENTIAL REPORT NO. 077
 OCCURRENCE CODE 03
 REPORT TYPE L
 REVISION NO. 0

ACTION TAKEN X18
 FUTURE ACTION G19
 EFFECT ON PLANT B20
 SHUTDOWN METHOD Z21
 HOURS 0000
 ATTACHMENT SUBMITTED Y23
 NPRO-4 FORM SUB. N24
 PRIME COMP. SUPPLIER N25
 COMPONENT MANUFACTURER Y01026

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

10 LT-N024A-2 and N024B-2 saw a false low level when a momentary pressure spike occurred  
11 due to the opening of the LT-N025A variable sensing leg root isolation valve. The  
12 ATWS/RPT signal was reset and both pumps were returned to service within six minutes  
13 of the event. By September 20, 1983, PT-55.3PC will be revised to reduce the  
14 potential of future similar events.

FACILITY STATUS E28
 % POWER 08729
 OTHER STATUS NA30
 METHOD OF DISCOVERY B31
 DISCOVERY DESCRIPTION Periodic Test32

ACTIVITY CONTENT RELEASED OF RELEASE Z33
 AMOUNT OF ACTIVITY NA35
 LOCATION OF RELEASE NA36

PERSONNEL EXPOSURES NUMBER 00037
 TYPE Z38
 DESCRIPTION NA39

PERSONNEL INJURIES NUMBER 00040
 DESCRIPTION NA41

LOSS OF OR DAMAGE TO FACILITY TYPE Z42
 DESCRIPTION NA43

PUBLICITY ISSUED DESCRIPTION N44

8309230302 830914  
 PDR ADOCK 05000324  
 S PDR

NRC USE ONLY

NAME OF PREPARER M. J. Pastva, Jr.

PHONE 919-457-9521

LER ATTACHMENT - RO #2-83-77

Facility: Unit No. 2

Event Date: August 15, 1983

During the performance of the loop calibration of the Unit No. 2 remote shutdown panel reactor vessel level monitoring instrumentation, PT-55.3PC, as reactor vessel level transmitter 2-B21-LT-N026A was being returned to service, both reactor recirculation pumps tripped due to the actuation of ATWS/RPT instruments 2-B21-LT-N024A-2 and N024B-2. All three of these instruments share a common reactor reference leg and variable sensing leg.

This event occurred as a result of a momentary pressure spike in the common variable sensing leg of these instruments, which caused the LT-N024A-2 and N024B-2 to see a false low level and actuate. When the low pressure instrument root valve of LT-N026A was opened, the subject pressure spike occurred. Within six minutes of this event, the ATWS/RPT signal was reset and both reactor recirculation pumps were returned to service. N026A Yarway, Model No. 4418C, was then returned to service.

As a result of this event, the PT will be revised to provide for utilizing the recirculation system bypass/test switches and improved sequence of valving and repressurizing instrumentation in order to prevent inadvertent recirculation pump trips. The expected date of completion for these changes to the PT procedure is September 20, 1983.

**CP&L**

83 SEP 20

AH: 25  
Carolina Power & Light Company

Brunswick Steam Electric Plant  
P. O. Box 10429  
Southport, NC 28461-0429  
September 14, 1983

FILE: B09-13510C  
SERIAL: BSEP/83-3072

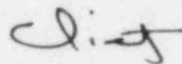
Mr. James P. O'Reilly, Administrator  
U. S. Nuclear Regulatory Commission  
Region II, Suite 3100  
101 Marietta Street N.W.  
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-324  
LICENSE NO. DPR-62  
LICENSEE EVENT REPORT 2-83-77

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 2, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,



R. R. Dietz, General Manager  
Brunswick Steam Electric Plant

RMP/clh/LETJ03

Enclosure

cc: Mr. R. C. DeYoung  
NRC Document Control Desk

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