AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	Fort Calhoun Station
DATE	August 6, 1991
COMPLETED BY	M.L. Edwards
TELEPHONE	(402)636-2451

MONTH July 1991

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	461	17	463
2	462	18	461
3	464	19	461
4	465	20	461
5	466	21	461
6	465	22	460
7	463	23	461
8	465	24	464
9	467	25	465
10	470	26	466
11	470	27	467
12	468	28	470
13	467	29	470
14	467	30	470
15	467	31	468
16	466		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE August 6, 1991
COMPLETED BY M.L. Edwards
TELEPHONE (402)636-2451

OPERATING STATUS

1. Unit Name: Fort Calhoun Station — Notes
2. Reporting Period: July 1991
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 502
7. Maximum Dependable Capacity (Net MWe): 478
8. If changes occur in Capacity Ratings (Item Numbers 3 through 7) Since Last Report, Give Reasons:
   N/A
9. Power Level to Which Restricted, If Any (Net MWe): N/A
10. Reasons for Restrictions, If Any: N/A

		This Month	Yr-to-Date	Cumulative
11.	Hours in Reporting Period	744.0	5,087.0	156,457.0
12.	Number of Hours Reactor was Critical	744.0	4,907.0	121,695.7
13.	Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14.	Hours Generator On-Line	744.0	4,896.2	120,326.1
15.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated (MWH)	1,111,316.9	5,986,074.5	157,269,563.8
17.	Gross Electrical Energy Generated (MWH)	363,156.0	1,967,788.0	51,717,914.2
18.	Net Electrical Energy Generated (MWH)	346,100.1	1,860,541.7	49,345,318.0
19.	Unit Service Factor	100.0	96.2	76.9
20.	Unit Availability Factor	100.0	96.2	76.9
21.	Unit Capacity Factor (Using MDC Net)	97.3	76.5	68.3
22.	Unit Capacity Factor (Using DER Net)	97.3	76.5	66.8
23.	Unit Forced Outage Rate	0.0	3.8	3.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    NONE
25. If Shut Down at End of Report Period, Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

	Forcast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY	N/A	
COMMERCIAL OPERATION		

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending July 1991

1. Scheduled date for next refueling shutdown. — January 31, 1992

2. Scheduled date for restart following refueling. — May 1, 1992

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be? — Incorporate specific requirements resulting from reload safety analysis.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. — N/A

   c. If no such review has taken place, when is it scheduled? — N/A

4. Scheduled date(s) for submitting proposed licensing action and support information. — November 1991

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. — New fuel supplier
   New LOCA Analysis

6. The number of fuel assemblies:
   a) in the core — 133 Assemblies
   b) in the spent fuel pool — 477 Assemblies
   c) spent fuel pool storage capacity — 729 Assemblies
   d) planned spent fuel pool storage capacity — Planned to be increased with higher density spent fuel racks.

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1995 *

* Capability of full core offload of 133 assemblies lost

Prepared by Kevin Holtt Date 8-6-91

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE August 8, 1991
COMPLETED BY M. L. Edwards
TELEPHONE (402) 636-2451

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH JULY 1991

No.	Date	Type (1)	Duration (Hours)	Reason (2)	Method of Shutting Down Reactor (3)	Licensee Event Report #	System Code (4)	Component Code (5)	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns or significant reductions in power during the month of July 1991.

1
F-Forced
S-Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

(9/77)

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

July 1991
Monthly Operating Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at a nominal 100% power through the month of July 1991.

The following NRC inspections took place in July:

IR 91-10 Training Program
IR 91-19 Liquid/Gaseous/Solid Rad Waste & Transportation

The following LERs were submitted:

LER-91-10	Auxiliary Steam Piping in Room 57 Outside Design Basis (HELB)
LER-91-11	Pressurizer Pressure Low Signal (PPLS) Setpoints
LER-91-12	Emergency Diesel Generator Auto-start Due to Loss of Transformer (T1A-4)
LER-91-13	Diesel Generator Auto-start Due to ST on DG-1
LER-91-14	Radiation Monitor RM-054A Out of Service

A. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

None

B. RESULTS OF LEAK RATE TESTS

The results of the Reactor Coolant Leak Rate tests for July 1991 indicate that the Reactor Coolant System (RCS) and Chemical and Volume Control System (CVCS) are both relatively leak tight. During July, the RCS total leak rate averaged 0.14 gpm.

The month's maximum leak rate was recorded on July 6, when the total leak rate was 0.293 gpm and the unknown leak rate was 0.228 gpm. Worn packing on charging pump CH-1B was the apparent cause. Charging pump CH-1B was repacked and the leak rate returned to normal. At the end of July, the leak rate increased again. Worn packing on charging pump CH-1A is suspected to be the cause of this recent increase in the leak rate.

C. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10 CFR 50.59

Amendment No.	Description
None	

D. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF JULY 1991

The 125 VDC battery number one (EE-8A) developed a crack on the corner of cell #15 causing an electrolyte leak. After the crack was temporarily repaired with clear silicone RTV caulk and the cell was filled with distilled water, maintenance was initiated to replace the cell.

Raw water pump AC-10D was removed, a new pump bowl assembly was installed and the pump was reassembled. The pump internals showed normal wear. Post maintenance test results were satisfactory and new baseline data was recorded.

During monthly surveillance testing on reactor protective system (RPS) trip unit B/TU-08, the setpoints were found within the acceptance range but close to the upper limit. The pre-trip and trip setpoints were readjusted and the surveillance test was performed again to verify operability.

The sample pump for containment stack radiation monitor, RM-061, was degrading in performance as indicated by low flow and vacuum readings. The sample pump was replaced. Containment stack radiation monitor, RM-050, was cycling from 10,000 to 1,000 to 10,000 cpm. Troubleshooting identified the ratemeter as the cause of the cycling irregularity and it was replaced. Other maintenance activities on RM-050 included repair of the sample flow fault light and the power available light. Two fuses were found blown due to a seized solenoid. Repairs included the replacement of the filter advance switch, filter advance solenoid, the rectifier and diode.

Replaced piping, fittings and hoses on iodine stack monitor RM-060 and installed seismic support for RM-060 flow totalizer.

During monthly testing the radiator exhaust damper would not open on diesel generator number two (DG-2) because the dampers were binding on the outside. DG-2's dampers were adjusted and a broken roll pin in the linkage mechanism was replaced. The diesel generator number one (DG-1) radiator exhaust damper was inspected to verify that the damper moved freely. DG-1's lower right damper blade was damaged and the roll pin was cracked. Repairs to DG-1 included straightening the edge of the damper and replacement of the roll pin. Post maintenance testing was performed on each diesel generator and the dampers were cycled to ensure there was no binding. Each diesel generator was started and operability testing verified the dampers opened as required.

The handwheel gear drive key on the inner door of the personnel air lock sheared, causing failure of the inner lock. Repairs included disassembly of the drive mechanism, replacement of the key, reassembly and adjustment of the interlocks. Aging and cycle fatigue is the suspected cause of failure.