

UPDATE REPORT - Previous Report Date 07/01/83

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 NYIPS 200-000000-000341114 (5)
7 8 9 14 15 25 26 30 57 CAT 58

CON'T
01 REPORT SOURCE L60500024706018309018309
7 8 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During normal operation, Flow Control Valve No. 406A, on Aux. Feedwater
03 Pump No. 21 discharge line to Steam Generator No. 21 did not open when
04 required during a surveillance test (Tech. Spec. 3.4.A(4)). An alternate
05 flow path supplied by Aux. Feedwater Pump No. 22 was verified available.
06 The health and safety of the public were unaffected. There were no
07 previous similar events.

08

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
C H B B V A L V E X F G
9 10 11 12 13 14 15 16

17 LER/RO REPORT NUMBER 83 025 03 X 1
21 22 23 24 25 26 27 28 29 30 31 32

ACTION FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
A Z Z Z 0000 Y Y N C 635
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Upon disassembly, the cage of FCV-406A was found chipped. The cage and
11 associated gaskets were replaced with like and kind. No foreign material
12 was found in pipe or valve. Valve was tested satisfactorily and returned
13 to service within allowed time. FCV-406A is a 2" - 900 psi Copes Vulcan
14 diaphragm operated Globe Valve.

15 FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION
E 100 NA B Surveillance Test
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

16 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE
Z Z NA NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

17 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION
000 Z NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

18 PERSONNEL INJURIES NUMBER DESCRIPTION
000 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION
Z NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

20 PUBLICITY ISSUED DESCRIPTION
Z NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

Gary Hinrichs

PHONE: 914-526-5548

8309130102 830901
PDR ADOCK 05000247
S PDR

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ATTACHMENT

Docket No. 50.247
LER 83-025/03L-0

Consolidated Edison of New York, Inc.
Indian Point Station, Unit 2

On Wednesday, June 1, 1983, while performing the Auxiliary Boiler Feed Pumps Discharge Valves Functional Test, Flow Control Valve (FCV)-406A did not open when required (T.S.3.4.A (4)). The Auxiliary Feedwater System supplies high pressure feedwater to the steam generators to maintain a water inventory for removal of decay heat from the Reactor Coolant System. This system consists of a turbine driven pump (No. 22) and two motor driven pumps (No. 21 and 23). FCV-406A is in the line from No. 21 Auxiliary Boiler Feed Pump to Steam Generator No. 21. A redundant flow path to Steam Generator No. 21 supplied by Pump No. 22, was verified operable.

The disassembly of FCV-406A found that the cylinder assembly or cage of FCV-406A was chipped. The cause is attributed to manufacturer's defect in the cage. No foreign material or debris were found in the valve or pipe. The cage was replaced with like and kind. The flexitallic gaskets were also replaced. The valve was tested satisfactorily and was returned to service within the time period allowed by Technical Specification 3.4.B.

FCV-406A is a 2" - 900 psi Copes Vulcan (Blaw-Knox) globe valve with a Model D-100-60 diaphragm operator.

John D. O'Toole
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, NY 10003
Telephone (212) 460-2533

September 1 1983

Re: Indian Point Unit No. 2
Docket No. 50-247
LER-83-025/03X-1

Dr. Thomas E. Murley,
Regional Administrator-Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pa. 19406

Dear Dr. Murley:

Transmitted herewith is an updated report for Licensee Event Report
LER-83-025.

Very truly yours,

Mr. L. L.
for J. D. O'Toole

Attach.
CC:

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. Thomas Foley, Senior Resident Inspector
U. S. Nuclear Regulatory Commission
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