



Northern States Power Company

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August 26, 1983

Director
Office of Nuclear Reactor Regulation
U S Nuclear Regulatory Commission
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

First Ten Year ISI/IST Program Completion
Unit 1 Request For Extension

Attached are forty copies of a request for relief related to the Unit 1 Inservice Inspection and Testing (ISI/IST) Program. This request is submitted in accordance with 10 CFR 50, Section 50.55a(g)(5)(iii). A fee determination form and check in the amount of \$4,000.00 is attached.

Northern States Power Company is submitting this request for time extension in completing the first ten year inservice inspection and testing program requirements. The two program requirements requiring extra time are:

1. ISI - inspection of the reactor vessel welds and areas requiring the use of the reactor vessel inspection tool or lower internals removed.
2. IST - performance of the hydrostatic tests for class 1, 2 and 3 equipment as noted on pages 1.2-1 of the IST program.

All the other requirements for Unit 1 in the ISI/IST program will be completed by the end of 1984.

Prairie Island Unit 1 ten year outage is scheduled to start January 5, 1985 and Unit 2 August 16, 1985. This schedule places the start date of Unit 1 ten year outage 29 days over the time allowed by ASME Code Section XI for completion of the inspection program. The licensee requests an extension of the grace period for the completion of the program until the 1985 outage for Unit 1 (Cycle 10). No extension is being requested at this time for Unit 2 because the outage is still within the ten years plus the allowed grace period of one year for the program. The licensee requests and needs the extension to be granted because:

*AO47
3/40 w/check
\$4,000*

NORTHERN STATES POWER COMPANY

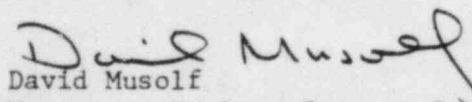
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1. The time extension interval requested is minimal and it is for only two areas of the program.
2. Those inspections needing the time extension are those which require the core to be unloaded and/or an extended outage to complete.
3. Monticello's outage to replace the recirculation piping is scheduled from January 28, 1984 to August 14, 1984. NSP does not schedule two large unit outages at the same time. If Prairie Island would do the ten year outage for Unit 1 in 1984, it would overlap with Monticello's outage causing two units to be out of service simultaneously. The average cost for one of the units to be out of service is approximately \$200,000 per day. The scheduling of a second unit outage at the same time would as a minimum double the cost per day. The current schedule has Unit 1 back on line before Monticello starts its' outage.
4. The available manpower resources would be exceeded by having two units in major outages at the same time.

The specific sections of the ISI/IST program for which extension is being requested are:

1. ISI - Items B1.1, B1.2, B1.3, B1.4, B1.6, B1.9, B1.12, B1.14, B1.15, B1.17, B1.19 and B4.1
2. IST - The ten year hydrotests as noted on page 1.2.1 of the ISI/IST manual.

Please contact us if you have any questions related to our request for relief from the requirements of our ASME Code Section XI Inservice Inspection and Testing Program.


David Musolf
Manager - Nuclear Support Services

cc: Regional Administrator-III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
G Charnoff

Attachment

REV 10

Instructions for Entering

Insert pages in the Prairie Island's ASME Code Section XI Inservice Inspection and Testing Program and Information Required for NRC Review of Requests for Relief from ASME Code Section XI Requirements as follows.

Remove Page

i
iii
1.1.1-2
1.1.1-3
1.1.1-4
1.1.1-5
1.1.1-6
1.1.1-14
1.2-1
1.5-48
-
-

Insert Attached Revised Pages

i
iii
1.1.1-2
1.1.1-3
1.1.1-4
1.1.1-5
1.1.1-6
1.1.1-14
1.2-1
1.5-48
1.5-49 (new)
1.5-49A (new)

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

DOCKET NOS. 50-282 LICENSE NOS. DPR-42
 50-306 DPR-60

ASME CODE SECTION XI INSERVICE INSPECTION AND TESTING PROGRAM

AND

INFORMATION REQUIRED FOR NRC REVIEW OF REQUESTS FOR RELIEF FROM
ASME CODE SECTION XI REQUIREMENTS

SUBMITTED: February 1, 1978

REVISED: Revision 1
September 15, 1978

Revision 2
June 8, 1979

Revision 3
September 19, 1979

Revision 4
April 17, 1980

Revision 5
September 3, 1980

Revision 6
July 31, 1981

Revision 7
December 23, 1981

Revision 8
April 19, 1983

Revision 9
June 24, 1983

Revision 10
August 26, 1983

RECORD OF REVISIONS

<u>Page No</u>	<u>Revision No</u>	<u>Page No</u>	<u>Revision No</u>
Cover	10	1.5-6	Original
ii	4	1.5-7	1
iii	10	1.5-7A	1
iv	9	1.5-7B	2
1-1	4	1.5-8 thru 1.5-9	Not Used
1.1.1-1	4	1.5-10	1
1.1.1-2 thru 1.1.1-6	10	1.5-11	Original
1.1.1-7	1	1.5-12	1
1.1.1-8 thru 1.1.1-10	Original	1.5-12A	2
1.1.1-11	6	1.5-13	Original
1.1.1-12 thru 1.1.1-13	Original	1.5-14 thru 1.5-15A	8
1.1.1-14	10	1.5-16	1
1.1.1-15 thru 1.1.1-17	Original	1.5-17	Original
1.1.1-18	1	1.5-18	1
1.1.1-19	Original	1.5-18A	1
1.1.1-20	1	1.5-19	Original
1.1.1-21 thru 1.1.1-25	Original	1.5-20	1
1.1.1-26 thru 1.1.1-28	6	1.5-21	Not Used
1.1.1-29 thru 1.1.1-33	Original	1.5-22 thru 1.5-23	Original
1.1.1-34	9	1.5-24	2
1.1.1-35	Original	1.5-25	6
1.1.1-36	6	1.5-26	Not Used
1.1.1-37 thru 1.1.1-39	Original	1.5-27 thru 1.5-28	Original
1.1.2-1	4	1.5-29	8
1.1.2-2 thru 1.1.2-27	6	1.5-30	Original
1.1.3-1	4	1.5-31	8
1.2-1	10	1.5-32 thru 1.5-34	1
1.3-1	4	1.5-34A	1
1.3-2	1	1.5-35	Original
1.4-1	4	1.5-36	6
1.4-2	1	1.5-37 thru 1.5-39	Original
1.4-3 thru 1.4-4	2	1.5-40	1
1.4-5	4	1.5-41	Original
1.4-6	1	1.5-42	1
1.4-7 thru 1.4-8	4	1.5-43	2
1.4-9	2	1.5-44 thru 1.5-46	Not Used
1.4-10	1	1.5-47	Original
1.4-11	7	1.5-48 thru 1.5-49	10
1.4-12	2	1.5-49A	10
1.4-13	Not Used	1.5-50	1
1.4-14 thru 1.4-15	2	1.5-51	Original
1.4-16	1	1.5-52 thru 1.5-55	6
1.4-17	6	1.5-56	3
1.4-18	4	1.5-57 thru 1.5-58	Original
1.4-19 thru 1.4-20	1	1.5-59	6
1.4-21	4	1.5-60	8
1.4-22	1	1.5-61 thru 1.5-62	Original
1.4-23	4	1.5-63	6
1.4-24 thru 1.4-34	1	1.5-64 thru 1.5-68	1
1.5-1	2	1.5-69	2
1.5-2	1	1.5-70	4
1.5-3	8		
1.5-3A	8		
1.5-4 thru 1.5-5	8		

NORTHERN STATES POWER CO.
PRAIRIE ISLAND UNIT 1

TEN YEAR INTERVAL INSPECTION SUMMARY

TABLE 1.1

PAGE 1 OF 5

MAJOR ITEM: REACTOR VESSEL

SUB ITEM	EXAM. CATE- GORY	COMPONENT OR SYSTEM, AND DESCRIPTION OF ITEM TO BE EXAMINED	TOTAL NO. PER ITEM	NDE METHODS	IDENTIFICATION	EXAMINATION AMOUNT AND EXTENT	INSPIC- TION PERIOD	LOCATION OR SYSTEM NUMBER	RUNNING PERCENT	REMARKS
B1.1	B-A	<u>LONGITUDINAL AND CIRCUMFERENCE- TIAL SHELL WELDS IN CORE REGION</u>								REQUIRES THE REMOVAL OF CORE BARREL RELIEF NO. 42
		LONGITUDINAL WELDS	NONE	---	---	---	---	---	---	
		CIRCUMFERENTIAL WELDS	1	U.T.	WELD NO. 3 (ISI-48)	21 FEET (MIN.)	THREE	LOWER SHELL ASSEMBLY TO INTERMEDIATE WELD	50	RELIEF NO. 55
B1.2	B-B	<u>LONGITUDINAL AND CIRCUMFERENCE- TIAL WELDS IN SHELL (OTHER THAN THOSE OF CATEGORY B-A AND B-C) AND MERIDIONAL AND CIRCUMFERENTIAL SEAM WELDS IN BOTTOM HEAD AND CLOSURE HEAD (OTHER THAN THOSE OF CATEGORY B-C)</u>								REQUIRES THE REMOVAL OF CORE BARREL RELIEF NO. 42
		LONGITUDINAL WELDS	NONE	---	---	---	---	---	---	
		MERIDIONAL WELDS	NONE	---	---	---	---	---	---	
		CIRCUMFERENTIAL WELDS	3	U.T.	WELD NO. 2 (ISI-48)	25 INCHES (MIN.)	THREE	UPPER SHELL ASSEMBLY TO LOWER SHELL ASSEM- BLY	5	RELIEF NO. 55
					WELD NO. 4 (ISI-48)	25 INCHES (MIN.)	THREE	LOWER TRANSITION HEAD TO SHELL WELD	5	
					WELD NO. 5 (ISI-48)	25 INCHES (MIN.)	THREE	BOTTOM HEAD RING TO LOWER TRANSITION HEAD WELD	5	
B1.3	B-C	<u>VESSEL-TO-FLANGE AND HEAD-TO- FLANGE CIRCUMFERENTIAL WELDS</u>								RELIEF NO. 55 & 42
		VESSEL TO FLANGE	1	U.T.	WELD NO. 1 (ISI-48)	CLOCKWISE, 14 FT (MIN.)	ONE	VESSEL FLANGE	33	
						CLOCKWISE, 12 FT. (MIN.)	TWO	VESSEL FLANGE	62	
						CLOCKWISE, 16 FT. (MIN.)	THREE	VESSEL FLANGE	100	

1.1.1.2

REVISION 10
8/26/83

NORTHERN STATES POWER CO.
PRAIRIE ISLAND UNIT 1

TABLE 1.1

TEN YEAR INTERVAL INSPECTION SUMMARY

PAGE 2 OF 5

MAJOR ITEM: REACTOR VESSEL

SUB ITEM	EXAM. CATEGORY	COMPONENT OR SYSTEM, AND DESCRIPTION OF ITEM TO BE EXAMINED	TOTAL NO. PER ITEM	TIME METHOD	IDENTIFICATION	EXAMINATION AMOUNT AND EXTENT	INSPECTION PERIOD	LOCATION OR SYSTEM NUMBER	RUNNING PERCENT	REMARKS
B1.4	B-D	HEAD TO FLANGE	1	U. T.	WELD NO. 6 (ISI-49)	CLOCKWISE, 14 FEET (MIN.) CLOCKWISE, 14 FEET (MIN.) CLOCKWISE, 14 FEET (MIN.)	ONE TWO THREE	HEAD FLANGE HEAD FLANGE HEAD FLANGE	33 66 100	RELIEF NO. 55 & 42 INSPECTION OF INLET NOZZLES AND SI NOZZLES REQUIRES REMOVAL OF CORE BARREL
		PRIMARY NOZZLE-TO-VESSEL WELDS AND NOZZLE INSIDE RADIUSED								
		REACTOR CORE COOLANT NOZZLES								
		OUTLET NOZZLES	2	U. T.	RCC-A-1 (ISI-50) RCC-B-1 (ISI-50)	1 WELD (100%) 1 WELD (100%)	ONE TWO	180° 0° (29-RC-1A) (29-RC-1B)	25 50	
		INLET NOZZLES	2	U. T.	RCC-A-14 (ISI-50) RCC-B-14 (ISI-50)	1 WELD (100%) 1 WELD (100%)	THREE THREE	120° 300° (27.5-RC-3A) (27.5-RC-3B)	75 100	
B1.5	B-E	SAFETY INJECTION NOZZLES	2	U. T.	LOOP A (ISI-30) LOOP B (ISI-30)	1 WELD (100%) 1 WELD (100%)	THREE THREE	260° 80° (4-RC-14A) (4-RC-14B)	50 100	RELIEF NO. 64
		VESSEL PENETRATIONS, INCLUDING CONTROL ROD DRIVE AND INSTRUMENTATION PENETRATIONS								RELIEF NO. 42
		CONTROL ROD PENETRATIONS	40	V		3 PENETRATIONS (MIN.) 3 PENETRATIONS (MIN.) 4 PENETRATIONS (MIN.)	ONE TWO THREE	TOP OF REACTOR VESSEL CLOSURE HEAD	9 15 25	PLANT OPERATIONS
		INSTRUMENTATION PENETRATIONS	36	V		3 PENETRATIONS (MIN.) 3 PENETRATIONS (MIN.) 3 PENETRATIONS (MIN.)	ONE TWO THREE	UNDER REACTOR VESSEL BOTTOM HEAD	8 17 25	PLANT OPERATIONS

NORTHERN STATES POWER CO.
PRAIRIE ISLAND UNIT 1

TEN YEAR INTERVAL INSPECTION SUMMARY

TABLE 1.1

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MAJOR ITEM: REACTOR VESSEL

SUB ITEM	EXAM. CATE-GORY	COMPONENT OR SYSTEM, AND DESCRIPTION OF ITEM TO BE EXAMINED	TOTAL NO. PER ITEM	NDE METHOD	IDENTIFICATION	EXAMINATION AMOUNT AND EXTENT	INSPEC-TION PERIOD	LOCATION OR SYSTEM NUMBER	RUNNING PERCENT	REMARKS
B1.6	B-F	REACTOR VESSEL HEAD VENT	1	V	I-RC-36 to RC-8-5	1 PENETRATION EXAMINATION IN ACCORDANCE WITH CATEGORY B-P		1-RC-36 TO RC-8-5		NOZZLE-TO-SAFE END WELD AND SAFE END-TO-PIPE WELD (B4.1) EXAMINE AS ONE. (SAND PLUG REMOVAL REQ.) RELIEF NO. 55 & 42
		NOZZLE-TO-SAFE END WELDS			(ISI-50)					
		REACTOR CORE COOLANT NOZZLES								
		OUTLET NOZZLE SAFE END WELDS	2	S-U.T.	RCC-A-1S.E. RCC-B-1S.E.	1 WELD (100%) 1 WELD (100%)	ONE TWO	180° 0° (29-RC-1A) (29-RC-1B)	25 50	
		INLET NOZZLE SAFE END WELDS	2	S-U.T.	RCC-A-14S.E. RCC-B-14S.E.	1 WELD (100%) 1 WELD (100%)	THREE THREE	120° 300° (27.5-RC-3A) (27.5-RC-3B)	75 100	
		REACTOR VESSEL SAFETY IN-JECTION NOZZLES	2	S-U.T.	(ISI-30)					
		NOZZLE A SAFE END WELD			WELD NO. 1 S.E.	1 WELD (100%)	ONE	260° (4-RC-14A)	50	
		NOZZLE B SAFE END WELD			WELD NO. 1 S.E.	1 WELD (100%)	THREE	80° (4-RC-14B)	100	
B1.8	B-G-1	CLOSURE STUDS AND NUTS	48	S-U.T.	1 THRU 16 17 THRU 31 & 33 32 & 34 thru 48 (ISI-37)	16 (100%) 16 (100%) 16 (100%)	ONE TWO THREE	0° THRU 120° 120° THRU 240° 240° THRU 360°	33 66 100	RELIEF NO. 43
B1.9	B-G-1	LIGAMENTS BETWEEN THREADED STUD HOLES	48	U.T.	4 THRU 14 & 2 THRU 29 16, 17 & 32 THRU 42 REMAINDER (ISI-48)	16 (100%) 13 (100%) 19 (100%)	ONE TWO THREE	VESSEL FLANGE VESSEL FLANGE VESSEL FLANGE	33 60 100	RELIEF NO. 55 & 42 CLOSURE HEAD REMOVED
B1.10	B-G-1	CLOSURE WASHERS AND BUSHINGS			(ISI-37)					
		WASHERS	48PRS.	V	1 THRU 16 17 THRU 31 & 33 32 & 34 thru 48	16 PAIRS (100%) 16 PAIRS (100%) 16 PAIRS (100%)	ONE TWO THREE		33 66 100	
		BUSHINGS	NONE							

NORTHERN STATES POWER CO.
PRAIRIE ISLAND UNIT 1

TEN YEAR INTERVAL INSPECTION SUMMARY

TABLE 1.1

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MAJOR ITEM: REACTOR VESSEL

SUB ITEM	EXAM. CATEGORY	COMPONENT OR SYSTEM, AND DESCRIPTION OF ITEM TO BE EXAMINED	TOTAL NO. PER ITEM	INSPECTION METHOD	IDENTIFICATION	EXAMINATION AMOUNT AND EXTENT	INSPECTION PERIOD	LOCATION OR SYSTEM NUMBER	RUNNING PERCENT	REMARKS
B1.11	B-G-2	<u>PRESSURE RETAINING BOLTING</u>	9	V	3 COMOSEAL BOLTS PER WARMON CLAMP (3) ISI-38	3 BOLTS (100%) 3 BOLTS (100%) 3 BOLTS (100%)	ONE TWO THREE	CLOSURE HEAD, 120° CLOSURE HEAD, 240° CLOSURE HEAD, 360°	33 66 100	
B1.12	B-H	<u>INTEGRALLY WELDED VESSEL SUPPORTS</u>	2	U.T.	SUPPORT LUGS A&B ISI-50	2 LUGS (100%)	THREE	LUG A-60°, LUG B-240°	100	RELIEF NO. 54 AND 55 RELIEF NO. 42
B1.13	B-I-1	<u>CLOSURE HEAD CLADDING</u>	6	V-S.	HCP-1 & HCP-2 HCP-3 & HCP-4 HCP-5 & HCP-6	TWO, 36SQ. IN. AREAS TWO, 36SQ. IN. AREAS TWO, 36SQ. IN. AREAS	ONE TWO THREE	INTERIOR OF CLOSURE HEAD	— — —	RELIEF NO. 51
B1.14	B-I-1	<u>VESSEL CLADDING</u>	6	V	VCP-1&VCP-2 VCP-3&VCP-4 VCP-5&VCP-6	TWO, 36SQ. IN. AREAS TWO, 36SQ. IN. AREAS TWO, 36SQ. IN. AREAS	ONE TWO THREE	IRRADIATION SPECIMEN ACCESS OPENINGS	— — —	RELIEF NO. 42
B1.15	B-N-1	<u>VESSEL INTERIOR</u> UPPER INTERNALS AND LOWER INTERNALS	—	V	UPPER INTERNALS LOWER INTERNALS THERMAL SHIELD FLEXURES DRIVE RODS	EXAM REPRESENTATIVE REGIONS OF THOSE INTERIOR SURFACES AND INTERNALS MADE ACCESSIBLE BY THE REMOVAL OF COMPONENTS DURING NORMAL REFUELING OPERATION.	ONE TWO THREE		— — —	AREA BELOW CORE INSPECTED WHEN CORE BARREL IS REMOVED DURING THE THIRD PERIOD RELIEF NO. 42
B1.16	B-N-2	<u>INTERIOR ATTACHMENTS AND CORE SUPPORT STRUCTURES</u>	REMARKS	—	REMARKS					NOT APPLICABLE FOR PRESSURIZED WATER REACTOR VESSEL
B1.17	B-N-3	<u>REMOVABLE CORE-SUPPORT STRUCTURES</u>	—	V	CORE BARREL	100% OF THE VISUALLY ACCESSIBLE ATTACHMENT WELDS AND VISUALLY ACCESSIBLE SURFACES OF THE CORE SUPPORT STRUCTURE	THREE	CORE BARREL REMOVED FROM THE REACTOR VESSEL	—	RELIEF NO. 42

NORTHERN STATES POWER CO.
PRAIRIE ISLAND UNIT 1

TECH YEAR INTERVAL INSPECTION SUMMARY

TABLE 1.1

PAGE 5 OF 5

MAJOR ITEM: REACTOR VESSEL

SUB ITEM	EXAM. CATEGORY	COMPONENT OR SYSTEM, AND DESCRIPTION OF ITEM TO BE EXAMINED	TOTAL NO. PER ITEM	NDE METHODS	IDENTIFICATION	EXAMINATION AMOUNT AND EXTENT	INSPECTION PERIOD	LOCATION OR SYSTEM NUMBER	RUNNING PERCENT	REMARKS
81.18	B-0	CONTROL ROD DRIVE HOUSINGS PERIPHERAL CRD HOUSINGS	20	U.T.*	CR-37 AND CR-38	2 WELDS (100%)	THREE	TOP OF CLOSURE HEAD CR-37&CR-38 OPPOSITE CLOSURE STUD NO. 9 AND NO. 33	10	* R.T. MAY HAVE TO BE USED IN LIEU OF OR IN ADDITION TO U.T. BECAUSE OF THE I.D. CONFIGURATION OF THE CRD HOUSING WELDS MAY PREVENT ADEQUATE U.T. EXAMINATION.
81.19	B-P	EXEMPTED COMPONENTS	---	V	ALL COMPONENTS EXEMPTED FROM VOLUMETRIC AND SURFACE EXAMINATION BY IWB 1200*	ALL COMPONENTS SHALL BE EXAMINED IN ACCORDANCE WITH IWA-5000* DURING EACH SYSTEM LEAKAGE TEST AND EACH SYSTEM HYDROSTATIC TEST REQUIRED BY IWB-5000*	---	---	---	*REFERENCES TO THE ASME BOILER & PRESSURE VESSEL CODE, SECTION XI 1974 EDITION THRU AND INCLUDING SUMMER 1975 ADDENDA RELIEF NO 42

NORTHERN STATES POWER CO.
PRAIRIE ISLAND UNIT 1

TEN YEAR INTERVAL INSPECTION SUMMARY

TABLE 1.4

PAGE 1 OF 19

MAJOR ITEM: PIPING PRESSURE BOUNDARY

SUB ITEM	EXAM. CATE- GORY	COMPONENT OR SYSTEM, AND DESCRIPTION OF ITEM TO BE EXAMINED	TOTAL NO. PER ITEM	NDE METHODS	IDENTIFICATION	EXAMINATION AMOUNT AND EXTENT	INSP- ECTION PERIOD	LOCATION OR SYSTEM NUMBER	RUNNING PERCENT	REMARKS
B4.1	B-F	<u>SAFE-END TO PIPING AND SAFE- END IN BRANCH PIPING WELDS</u>								RELIEF NO. 46 RELIEF NO. 56 RELIEF NO. 42
		<u>REACTOR VESSEL</u>	(6)							
		REACTOR CORE COOLANT SYSTEMS	4	S-UT	RCC-A-1 S.E. RCC-B-1 S.E. RCC-A-14 S.E. RCC-B-14 S.E.	1 WELD 100% 1 WELD 100% 1 WELD 100% 1 WELD 100%	ONE TWO THREE THREE	180° 0° 120° 300° (29-RC-1A) (29-RC-1B) (27.5-RC-3A) (27.5-RC-3B)	25 50 75 100	
		SAFETY INJECTION SYSTEMS	2	S-UT	LOOP A, WELD 2 S.E. LOOP B, WELD 2 S.E.	1 WELD 100% 1 WELD 100%	ONE THREE	260° 80° (4-RC-14A) (4-RC-14B)	50 100	
		<u>STEAM GENERATOR NO. 11</u>	(2)							
		REACTOR CORE COOLANT SYSTEM	2	S-UT	RCC-A-5 S.E. RCC-A-4 S.E.	1 WELD 100% 1 WELD 100%	ONE THREE	CUBICLE A (31-RC-2A) CUBICLE A (29-RC-1A)	50 100	
		<u>STEAM GENERATOR NO. 12</u>	(2)							
		REACTOR CORE COOLANT SYSTEM	2	S-UT	RCC-B-5 S.E. RCC-B-4 S.E.	1 WELD 100% 1 WELD 100%	TWO THREE	CUBICLE B (31-RC-2B) CUBICLE B (29-RC-1B)	50 100	
		<u>PRESSURIZER</u>	(5)							
		SAFETY LINES	2	S-UT	8010A, 1 S.E. 8010B, 1 S.E.	1 WELD 100% 1 WELD 100%	ONE ONE	TOP OF PZR (6-RC-20A) TOP OF PZR (6-RC-20B)	20 40	
		SURGE LINE	1	S-UT	WELD 6, S.E.	1 WELD 100%	TWO	BOT OF PZR (10-RC-4)	60	
		RELIEF LINE	1	S-UT	WELD 1, S.E.	1 WELD 100%	THREE	TOP OF PZR (3-RC-21)	80	
		SPRAY LINE	1	S-UT	WELD 29, S.E.	1 WELD 100%	THREE	TOP OF PZR (3-RC-5) (4-RC-5)	100	

1.1.1-14

REVISION 10
8/26/83

ASME Section XI Pressure Testing Program - Unit No. 1 and Common Components

ASME Code Edition and Addenda: 1974 Edition through and including Summer 1975 Addenda

Program Period: April 16, 1977 to December 16, 1983

The system Code Class boundaries are established on the attached ASME Code Classification Drawings, Sheets 2 through 40. The Pressure Test Program for the Class 1, 2, and 3 systems is as follows:

ASME CODE CLASS	TEST TYPE	TEST FREQUENCY	TEST SPECIFICATION	REQUEST FOR RELIEF
1	Leakage	Refueling	IWB-5210(a) IWB-5221 IWA-5000	#20, #60
1	Hydrostatic	10 years	IWB-5210(b) IWB-5222 IWA-5000	#20, #42
2	Pressure Test	3 1/3 years	IWC-2412 IWC-2510 IWC-5210	#19, #20, #29
2	Hydrostatic Test	10 years	IWC-2412 IWC-2510 IWC-5210	#19, #20, #29, #42
3	Pressure	10 years	IWD-2410(b)	#20, #28, #30, #31 #42

40. REQUEST FOR RELIEF

COMPONENT	FUNCTION	ASME	
		Code Class	Vlv Cat
12,22, Diesel Cooling Water Pump	Remove heat from components that must function during accident conditions.	3	-

CODE REQUIREMENT

Inlet pressure will not be used to evaluate the condition of the pump as required by IWP-3.10 and 3210.

BASIS

The pump suction is located in the cooling water intake bay and suction pressure indication is not available.

ALTERNATE INSPECTION (testing)

Intake bay level will be factored into the differential pressure data used in the pump test.

SCHEDULE FOR IMPLEMENTATION

April 16, 1977

42. REQUEST FOR RELIEF

COMPONENT OR ITEM	CODE CLASS	PROGRAM TABLE	CODE ITEM	EXAM CATEGORY
LONGITUDINAL AND CIRCUMFERENTIAL SHELL WELDS IN CORE REGION.	1	1.1	B1.1	B-A
LONGITUDINAL AND CIRCUMFERENTIAL WELDS IN SHELL (OTHER THAN THOSE OF CATEGORY B-A AND B-C AND MERIDIONAL AND CIRCUMFERENTIAL SEAM WELDS IN BOTTOM HEAD AND CLOSURE HEAD (OTHER THAN THOSE OF CATEGORY B-C)).	1	1.1	B1.2	B-B.
VESSEL-TO-FLANGE AND HEAD-TO-FLANGE CIRCUMFERENTIAL WELDS.	1	1.1	B1.3	B-C
PRIMARY NOZZLE-TO-VESSEL WELDS AND NOZZLE INSIDE RADIUS.	1	1.1	B1.4	B-D
VESSEL PENETRATIONS, INCLUDING CONTROL ROD DRIVE AND INSTRUMENTION PENETRATIONS.	1	1.1	B1.5	B-E
NOZZLE-TO-SAFE END WELDS.	1	1.1	B1.6	B-F
LIGAMENTS BETWEEN THREADED STUD HOLES.	1	1.1	B1.9	B-G-1
INTEGRALLY WELDED VESSEL SUPPORTS.	1	1.1	B1.12	B-H
VESSEL CLADDING.	1	1.1	B1.14	B-I-1
VESSEL INTERIOR.	1	1.1	B1.15	B-N-1
REMOVABLE CORE-SUPPORT STRUCTURES.	1	1.1	B1.17	B-N-3
EXEMPTED COMPONENTS.	1	1.1	B1.19	B-P
SAFE-END TO PIPING AND SAFE-END IN BRANCH PIPING WELDS REACTOR VESSEL.	1	1.4	B4.1	B-F

42. REQUEST FOR RELIEF (Cont'd)

COMPONENT OR ITEM	CODE CLASS	PROGRAM PAGE	TEST FREQUENCY
HYDRO STATIC TESTS	1,2,& 3	1.2-1	10 years

CODE REQUIREMENT

These tests will be completed within the interval of 10 years plus 1 year allowed extension.

BASIS

The timing and scheduling to complete the Prairie Island Unit 1 first Ten-Year ISI/IST Program exceeds the eleven year time frame allowed by the Code. A detailed basis for this request is contained in NSP letter dated 8/26/83.

ALTERNATE

The tests will be done during the 10 year outage for Unit 1 schedule to start in January 1985.

SCHEDULE FOR IMPLEMENTATION

Three months from the submittal date of August 26, 1983.

1.5-49A

REV 10
8/26/83