

CONTROL BLOCK

REASON FOR NOT REPORTING (SEE INSTRUCTIONS)

01 ALBRF3 200-000000-00 41111 4

CONT

01 REPORT SOURCE L 605000296 080883 0 90683 7

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (6)

02 During normal operation on unit 3, with the unit operating at 95% power
03 the indication on LI-64-54A decreased 1.8 inches. The indication on
04 LI-64-66A (redundant indicator) did not change as there was no actual change
05 in torus water level. A 30 day LCO was entered per T.S. 3.2.F. There was
06 no danger to the health or safety of the public because redundant
07 indicator LI-64-66 remained operable and torus water level remained within
08 acceptable limits.

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP SUBCODE	VALVE SUBCODE
09 I	10 D	11 E	12 G	13 I	14 Z
INSTRU					
LEAD REPORT NUMBER	EVENT YEAR	SEQUENTIAL REPORT NO	OCCURRENCE CODE	REPORT TYPE	REVISION NO
17 8	18 3	19 049	20 03	21 L	22 0
ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED
23 A	24 Z	25 Z	26 Z	27 0000	28 Y
NEEDS	PRIME COMP SUPPLIER	COMPONENT MANUFACTURER			
29 N	30 L	31 G080			

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The power supply (PX-64-54) in the 64-54 instrument loop had become
11 unstable. The G.E. model 570012FAAC1 power supply was replaced, and the
12 loop was calibrated, functionally tested and returned to service. This
13 is considered a random event and no further recurrence control is planned.

FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
15 E	16 095	17 N/A	18 A	19 Engineer observed
ACTIVITY CONTENT RELEASED OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE		
20 Z	21 Z	22 N/A		
PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION		
23 000	24 Z	25 N/A		
PERSONNEL INJURIES NUMBER	DESCRIPTION			
26 000	27 N/A			
LOSS OF OR DAMAGE TO FACILITY TYPE	DESCRIPTION			
28 Z	29 N/A			
PUBLICITY ISSUED DESCRIPTION				
30 N	31 N/A			

8309120228 830906
PDR ADOCK 05000296
S PDR

NR USE ONLY

NAME OF PREPARED G. T. Chambers

PHONE (205)729-0841 IE22

LER SUPPLEMENTAL INFORMATION

BFRO-50- 296 / 83049 Technical Specification Involved 3.2.F
Reported Under Technical Specification 6.7.2.b.(2) Date Due NRC 09/07/83

Event Narrative:

Unit 1 was in a refueling outage. Units 2 and 3 were at 96% and 95% power respectively. Only unit 3 was affected by this event. During normal operation the indication on LI-64-54A decreased 1.8 inches with no actual torus water level change. Redundant torus water level indicator (LI-64-66) indicated no change in level and operations personnel verified that there was no leakage. A 30 day L.C.O. was entered per Tech. Spec. 3.2.F which requires both LI-64-54A & 66 to be operable. Upon investigation it was discovered that PX-64-54, the power supply for the instrument loop that feeds LI-64-54A, had become unstable. PX-64-54 was replaced, and the loop was calibrated, functionally tested and returned to service per Surveillance Instruction 4.2.F-7 (Suppression Chamber Water Level-Calibration and Functional Test). There was no danger to the health or safety of the public because redundant indicator LI-64-66 remained operable and torus water level remained within acceptable limits. This was a random event and no further recurrence control is required.

Previous Similar Events:

None

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRR

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401
1750 Chestnut Street Tower II

USNRC REGION 7
ATLANTA, GEORGIA

83 SEP 7 49:01

September 6, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

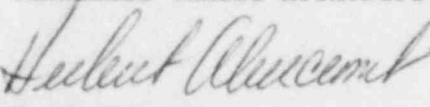
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET
NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE
REPORT BFRO-50-296/83049

The enclosed report provides details concerning a level indication for the
torus water level that gave a false indication. This report is submitted
in accordance with Browns Ferry unit 3 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY


H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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