

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

W. L. STEWART
VICE PRESIDENT
NUCLEAR OPERATIONS

December 30, 1982

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attn: Mr. Robert A. Clark, Chief
Operating Reactors Branch No. 3
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20535

Serial No.: 726
FR/GLD: gmj
Docket Nos.: 50-338
50-339
License Nos.: NPF-4
NPF-7

Gentlemen:

AMENDMENT TO OPERATING LICENSES NPF-4 AND NPF-7
NORTH ANNA POWER STATION UNIT NOS. 1 AND 2
PROPOSED TECHNICAL SPECIFICATION CHANGE

Pursuant to 10CFR50.90, the Virginia Electric and Power Company requests an amendment, in the form of changes to the Technical Specifications, to Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station Unit Nos. 1 and 2.

As discussed with members of the Staff in meetings on August 11, 1981 and June 9, 1982, Vepco is engaged in a program to maximize the electrical output of North Anna Unit Nos. 1 and 2. Amendment No. 41 to Operating License NPF-4 allows operation of North Anna Unit No. 1 with a reactor coolant system (RCS) average temperature (Tavg) of 582.8°F. The proposed changes of this amendment will allow operation of the North Anna units with a Tavg of 587.8°F. The increase in Tavg will further increase secondary side steam pressure, resulting in greater plant electrical output.

Enclosures 1 and 2, respectively, provide Safety Evaluations for the NSSS and Balance of Plant (BOP) systems in support of this request. Several Chapter 15 FSAR safety analyses were performed to support plant operation with a Tavg of 587.8°F. The appendix to Enclosure 1 documents these analyses with revised FSAR pages for the affected transients. The accidents reanalyzed included a large break LOCA performed at the uprated conditions which resulted in an FQ limit of 2.20. Enclosure 4 provides Core Surveillance Reports associated with this limit for the current plant cycles, North Anna 1, Cycle 4 and North Anna 2, Cycle 2. The specific Technical Specifications changes required for this change are given in Enclosure 3. Several associated setpoint changes are required for operation at the increased RCS average temperature, and these will be incorporated into the Precautions, Limitations and Setpoints (PLS) document and the plant procedures upon approval of this request.

As noted in Enclosure 2, the BOP review determined no required plant modifications to support operation at the uprated conditions. The

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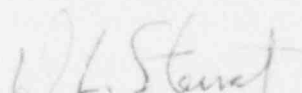
Unit No. 2 Mono-Ball support modification identified during the 2.5°F Tav_g increase review is designed to be acceptable for a Tav_g of 587.8°F. The BOP review did reveal a decrease in feedwater regulating valve operational flexibility at the uprated conditions. A change in valve trim is being investigated to provide improved valve performance.

This request has been reviewed and approved by the Station Nuclear Safety and Operating Committee and the Safety Evaluation and Control staff. It has been determined that this request does not involve any unreviewed safety questions as defined in 10CFR50.59.

We have evaluated this request in accordance with the criteria in 10CFR170.22. Since this request involves a safety issue which the staff should be able to determine does not involve a significant hazards consideration for Unit No. 1 and a duplicate safety issue for Unit No. 2, a Class III license amendment fee and a Class I license amendment fee is required for Unit No. 1 and Unit No. 2, respectively. A voucher check in the amount of \$4,400.00 is enclosed in payment of the required fee.

We are interested in implementing the 7.5°F increase in Tav_g on Unit No. 2 during the Spring 1983 refueling outage, which ends on May 15, 1983. This would provide an opportunity to perform any feedwater regulating valve trim changes which may be desired. Consequently, we wish to discuss this request with appropriate members of the Staff in January 1983. Please contact us if you need additional information to review this submittal.

Very truly yours,


W. L. Stewart

Enclosures

- (1) NSSS Safety Evaluation for Tav_g 587.8°F
- (2) BOP Safety Evaluation for Tav_g of 587.8°F
- (3) Proposed Technical Specifications Changes
- (4) Core Surveillance Reports for North Anna 1, Cycle 4 and
North Anna 2, Cycle 2
- (5) Voucher Check for \$4,400.00

cc: Mr. James P. O'Reilly
Regional Administrator
Region II
Atlanta, GA 30303

COMMONWEALTH OF VIRGINIA)
)
CITY OF RICHMOND)

The foregoing document was acknowledged before me, in and for the City and Commonwealth aforesaid, today by W. L. Stewart, who is Vice President-Nuclear Operations, of the Virginia Electric and Power Company. He is duly authorized to execute and file the foregoing document in behalf of that Company, and the statements in the document are true to the best of his knowledge and belief.

Acknowledged before me this 30th day of December, 19 82.

My Commission expires: 2-26, 19 85.

Ann C. Moore
Notary Public

(SEAL)

VIRGINIA ELECTRIC AND POWER COMPANY

CHECK VOUCHER
1578

BANK NO. 25

CHECK NO. 98670

DATE 12/27/82

VENDOR NO. 3301

LINE	DATE	INVOICE NO./OR DESCRIPTION	GROSS AMOUNT	DISCOUNT	NET AMOUNT
1	12/21/82	NA 1&2 TECH SPEC CHANGE	4,400.00	0.00	4,400.00

ATTACHED CHECK ISSUED AS PAYMENT OF ITEMS LISTED ABOVE - PLEASE DETACH STUB AND CASH CHECK PROMPTLY.

Vepco

BANK OF VIRGINIA

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA

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VOID IF NOT CASHED IN 90 DAYS

03301 25 0098670 00000440000

PAY FOUR THOUSAND FOUR HUNDRED AND 00/100 DOLLARS

TO
THE
ORDER
OF
**UNITED STATES NUCLEAR
REGULATORY COMMISSION
NUCLEAR REG C DC 20555**

BK	CHECK NO.	DATE	VENDOR NO.	AMOUNT
25	98670	12/27/82	3301	\$4,400.00

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