ATTACHMENT I

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50/395
UNIT V.C. Summer I
DATE 01-05-83
COMPLETED BY G.J. Taylor
TELEPHONE (803) 345-5209

MONTH DECEMBER 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	216
2	1	18	0
3	167	19	0
4	195	20	238
5	194	21	396
6	193	22	136
7	198	23	308
8	196	24	411
9	192	25	409
10	202	26	409
11	252	27	408
12	350	28	143
13	384	29	0
14	383	30	143
15	387	31	409
16	400		

ATTACHMENT II

OPERATING DATA REPORT

DOCKET NO. 50/395
UNIT V.C. Summer I
DATE 01-05-83
COMPLETED BY G. J. Taylor
TELEPHONE (803) 345-5209

OPERATING STATUS

1. REPORTING PERIOD: DECEMBER 1982 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2775 MAX. DEPEND. CAPACITY (MWe-Net): N/A
   DESIGN ELECTRICAL RATING (MWe-Net): 900
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): NA (50% MWth)
4. REASONS FOR RESTRICTION (IF ANY):
   The Operating License allows operations to 50% MWth for power operations testing.

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	651.1	1306.8	1306.8
6. REACTOR RESERVE SHUTDOWN HOURS	0	0	0
7. HOURS GENERATOR ON LINE	569.0	761.3	761.3
8. UNIT RESERVE SHUTDOWN HOURS	0	0	0
9. GROSS THERMAL ENERGY GENERATED (MWH)	651,071	795,873	795,873
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	185,650	216,217	216,217
11. NET ELECTRICAL ENERGY GENERATED (MWH)	166,357	191,196	191,196
12. REACTOR SERVICE FACTOR	N/A	N/A	N/A
13. REACTOR AVAILABILITY FACTOR	N/A	N/A	N/A
14. UNIT SERVICE FACTOR	N/A	N/A	N/A
15. UNIT AVAILABILITY FACTOR	N/A	N/A	N/A
16. UNIT CAPACITY FACTOR (Using MDC)	N/A	N/A	N/A
17. UNIT CAPACITY FACTOR (Using Design MWe)	N/A	N/A	N/A
18. UNIT FORCED OUTAGE RATE	N/A	N/A	N/A

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
    D-3 Steam Generator Modification -- March, 1983, thru May, 1983.
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	10-20-82	10-22-82
INITIAL ELECTRICITY	11-17-82	11-16-82
COMMERCIAL OPERATION	--	--

ATTACHMENT III

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50/395
UNIT NAME V.C. Summer I
DATE 01-05-83
COMPLETED BY G. J. Taylor
TELEPHONE (803) 345-5209

REPORT MONTH DECEMBER 1982

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
6	821129	F	46.7	G	3	6) Reactor trip due to low-low level in "B" Steam Generator while performing Steam Generator level test.
7	821217	F	62.5	G	3	7) Reactor trip due to hi-hi Feedwater Heater level, which caused turbine/ reactor trip.
8	821222	F	14.6	G	3	8) Reactor trip due to low-low level in Steam Generator "C", due to loss of Feedwater (Low Feedwater pressure signal).
9	821228	F	51.2	G	3	9) Reactor trip due to returning to service from a test, of the Solid State Protection System Multiplexer.

ATTACHMENT IV
NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Virgil C. Summer Nuclear Station Unit No. 1, has been in the process of continuing the power ascension test program to a 50% MWth power level. A 50% MWth power level was attained at approximately 1700 hours, December 12, 1982.

At 0745 hours, December 1, 1982, the unit tripped from a 2% power level. The trip was due to a low-low level in Steam Generator "A" while on emergency feedwater and attempting to raise power to 5% with the Main Steam Isolation Valves closed.

A reactor trip occurred at 1530 hours, December 1, 1982, from 0% power. The trip was due to a steam flow feed flow mismatch with a low level in Steam Generator "C".

A reactor trip occurred at 1326 hours, December 17, 1982, from 50% power. The trip was due to a Hi-Hi level in 6A Feedwater Heater, which caused a turbine/reactor trip. Work was being performed on the level controller, which caused a level deviation.

A reactor trip occurred at 0851 hours, December 22, 1982, from a 50% power level. The trip was due to a low-low level in Steam Generator "C". This low-low level was caused by low feedwater flow. Coincident with this reactor trip, pressurizer pressure indicated low which closed the pressurizer spray valves and energized the pressurizer heaters. This, combined with control rods being in manual, caused actual pressurizer pressure and level to increase so that the pressurizer Power Operated Relief Valves cycled several times. The event was caused by de-energization of the reactivity computer, which was monitoring points connected to the process control racks.

A reactor trip occurred at 0925 hours, December 28, 1982, from a 50% power level. The trip was caused by returning Train "A" of the Solid State Protection System multiplexer switch to the "A and B" position with B train reactor trip bypass breaker closed. This gave a General Warning alarm on both "A" and "B" trains of the Solid State Protection System, which in turn caused the reactor trip.

Virgil C. Summer Nuclear Station is presently operating at 50% power and continuing the power ascension test program.