

Omaha Public Power District
444 South 16th Street Mail
Omaha, Nebraska 68102-2247
402/636-2000

July 29, 1991
LIC-91-0093L

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, DC 20555

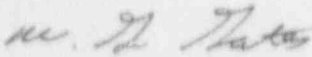
Reference: Docket No. 50-285

Gentlemen:

Subject: Licensee Event Report 91-14 for the Fort Calhoun Station

Please find attached Licensee Event Report 91-14 dated July 29, 1991. This report is being submitted pursuant to 10 CFR 50.73(a)(2)(i)(B). If you should have any questions, please contact me.

Sincerely,



W. G. Gates
Division Manager
Nuclear Operations

WGG/rkj

Attachment

c: R. D. Martin, NRC Regional Administrator
W. C. Walker, NRC Project Manager
R. P. Mullikin, NRC Senior Resident Inspector
INPO Records Center

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.5 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F630), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Fort Calhoun Station Unit No. 1

DOCKET NUMBER (2)

0 5 0 0 0 2 8 5 1 OF 0 3

PAGE (3)

TITLE (4)

Radiation Monitor RM-054A Out of Service

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)												
0	6	2	9	9	1	9	1	0	1	4	0	0	0	7	2	9	9	1	0	5	0	0	0
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 (Check one or more of the following) (11)																				
1																							
POWER LEVEL (10)			20.405(e) 50.73(a)(2)(iv) 73.71(b)																				
0.100			20.405(a)(1)(i) 50.73(a)(2)(v) 73.71(c)																				
			20.405(a)(1)(ii) 50.73(a)(2)(vi) OTHER (Specify in Abstract below and in Text, NRC Form 366A)																				
			20.405(a)(1)(iii) X 50.73(a)(2)(vii)(A)																				
			20.405(a)(1)(iv) 50.73(a)(2)(viii)(B)																				
			20.405(a)(1)(v) 50.73(a)(2)(ix)																				

LICENSEE CONTACT FOR THIS LER (12)

NAME

S. A. Lindquist, Shift Technical Advisor

TELEPHONE NUMBER

AREA CODE

4 0 2 5 3 3 - 6 8 2 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

YES (If yes, complete EXPECTED SUBMISSION DATE)

X NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 29, 1991 at 1659 hours, Fort Calhoun Station was operating at 100 percent power when the mode selector switch for steam generator blowdown radiation monitor RM-054A was discovered in the calibrate position. Upon discovery, the mode selector switch was returned to the operate position and steam generator A was sampled. For the approximately seventeen hours that the RM-054A mode selector switch was in the calibrate position, steam generator A blowdown radioactivity was not automatically recorded, nor was it manually recorded every four hours as required by Technical Specification 2.9.1(1)e. This violation of the Technical Specification is reportable pursuant to 10 CFR 50.73(a)(2)(i)(B).

The cause of this event was personnel error. During the performance of surveillance test OP-ST-SHIFT-0001 (Operations Technical Specification Required Shift Surveillance) on June 29, 1991 at 0020 by a senior reactor operator trainee, RM-054A's calibration was checked by taking the mode selector switch to the calibrate position. After the calibration check, the mode selector switch is required to be returned to the operate position. On this occasion, the mode selector switch was not returned to the operate position.

The corrective actions for this event consist of revising OP-ST-SHIFT-0001 to include a check-off when all radiation monitors being calibration checked are returned to operation and providing guidance on expectations for on-the-job training to both trainees and trainers.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (3)			PAGE (4)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Fort Calhoun Station Unit No. 1	0 5 0 0 0 2 8 5	9 1	— 0 1 4	— 0 0	0 2	OF 0 3

TEXT (If more space is required, use additional NRC Form 368A's)(17)

On June 29, 1991 at 1:59 hours, Fort Calhoun Station was operating at 100 percent power when the mode selector switch for steam generator blowdown radiation monitor RM-054A was discovered in the calibrate position. Upon discovery, the mode selector switch was returned to the operate position and steam generator A was sampled. For the approximately seventeen hours the RM-054A mode selector switch was in the calibrate position, steam generator A blowdown radioactivity was not automatically recorded nor was it manually recorded every four hours as required by Technical Specification 2.9.1(1)e. This violation of the Technical Specification is reportable pursuant to 10 CFR 50.73(a)(2)(i)(B).

The cause of this event was personnel error. During the performance of surveillance test OP-ST-SHIFT-0001 (Operations Technical Specification Required Shift Surveillance) on June 29, 1991 at 0020 by a senior reactor operator trainee, the RM-054A calibration was checked by taking the mode selector switch to the calibrate position. After the calibration check, the mode selector switch is required to be returned to the operate position. However, on this occasion the mode selector switch was not returned to the operate position. Because it was not known that RM-054A was in the calibrate position, thus being inoperable, the manual action necessary to cross connect RM-054B to monitor the other steam generator blowdown line was not taken.

The Fort Calhoun Station steam generator blowdown system is designed to provide redundant isolation and radiation monitoring capabilities for safe and proper operation if a primary to secondary system tube leak were to occur. Steam generator blowdown system radiation monitors RM-054A and RM-054B (for steam generators A and B, respectively) are designed to monitor and isolate both steam generator blowdown lines if either RM-054A or RM-054B detect high radiation in the main blowdown lines. These radiation monitors alarm in the control room on the first setpoint, as well as alarm and automatically isolate all blowdown lines at a second higher setpoint. RM-054A isolates the steam generator blowdown by means of valves HCV-1387A and HCV-1388A. Similarly, RM-054B isolates the steam generator blowdown by means of valves HCV-1387B and HCV-1388B. Process radiation monitor, RM-057 performs secondary system monitoring functions which alarm and thus alert the operator to isolate potential release paths to the environment. RM-057 monitors the condenser off-gas and will alarm and alert the operator to isolate the condenser off-gas from being released through a turbine building exhaust stack.

Based on the low steam generator sample activity and the fact that RM-054B and RM-057 did not alarm during this event, a safety concern did not exist. The results of the steam generator A sample indicated that the activity level was less than or equal to 1.88 E-6 microcuries per gram which is the lowest level detectable. Also, radiation monitors RM-054B and RM-057 were functioning properly and did not alarm during this event. Thus, Fort Calhoun Station had adequate means to detect and isolate a primary to secondary tube leak or rupture in one or both steam generators.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (3)			PAGE (3)	
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Fort Calhoun Station Unit No. 1	0 5 0 0 0 2 8 5 9 1	0 1 4	0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 306A's)(17)

Corrective Actions

The cause of this event was personnel error. The trainee performing the calibration check, failed to return RM-054A to operation. Since the trainee had successfully completed this test approximately five times prior to this event, it is not considered to be a trainee issue, but rather a procedural inadequacy. Therefore, the following actions will be taken: (1) OP-ST-SHIFT-0001 will be revised by October 15, 1991 to require a check-off after the radiation monitors being calibration checked are returned to operation, and (2) There will be added emphasis on the need for attention to detail, as both trainers and trainees will be provided with expectations on the performance of tasks during on-the-job training. This will be completed by October 21, 1991.

The following Licensee Event Reports involving the loss of steam generator blowdown radiation monitoring have been submitted: 74-03, 74-04, 75-21, and 82-11.