AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-311
Unit Name Salem # 2
Date August 10, 1983
Telephone 609-935-6000
Extension 4455

Completed by L. K. Miller

Month July 1983

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	0	16	0
2	0	17	0
3	0	18	0
4	0	19	0
5	0	20	0
6	0	21	0
7	0	22	0
8	0	23	0
9	0	24	0
10	0	25	0
11	0	26	0
12	0	27	0
13	0	28	0
14	0	29	0
15	0	30	0
		31	304

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OPERATING DATA REPORT

Docket No. 50-311
Date August 10, 1983
Telephone 935-6000
Extension 4455

Completed by L. K. Miller

Operating Status

1. Unit Name	Salem No. 2	Notes
2. Reporting Period	July 1983	
3. Licensed Thermal Power (MWt)	3411	
4. Nameplate Rating (Gross MWe)	1162	
5. Design Electrical Rating (Net MWe)	1115	
6. Maximum Dependable Capacity (Gross MWe)	1149	
7. Maximum Dependable Capacity (Net MWe)	1106	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	5087	15793
12. No. of Hrs. Reactor was Critical	155.1	625.4	11081.0
13. Reactor Reserve Shutdown Hrs.	0	3.3	38.9
14. Hours Generator On-Line	32.6	492.0	10829.2
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	53664	1163047.2	31679596.2
17. Gross Elec. Energy Generated (MWH)	9300	266640	10282890
18. Net Elec. Energy Generated (MWH)	(8646)	213907	9787554
19. Unit Service Factor	4.4	9.7	68.6
20. Unit Availability Factor	4.4	9.7	68.6
21. Unit Capacity Factor (using MDC Net)	0	3.8	56.0
22. Unit Capacity Factor (using DER Net)	0	3.8	55.6
23. Unit Forced Outage Rate	87.3	35.2	5.3

24. Shutdowns scheduled over next 6 months (type, date and duration of each) N/A

25. If shutdown at end of Report Period, Estimated Date of Startup: N/A

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	6/30/80	8/2/80
Initial Electricity	9/1/80	6/3/81
Commercial Operation	9/24/81	10/13/81

8-1-7.R2

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH JULY 1983

Docket No. 50-311
Unit Name Salem No.2
Date August 10, 1983
Telephone 609-935-6000
Extension 4455

Completed by L.K. Miller

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
83-048	5/28	F	840.0	B	1	---	WA	PIPEXX	Nuclear Service Water Problems other
83-050	7/02	"	24.0	"	"	"	"	"	Nuclear Residual Heat Removal/ Decay Heat
83-052	7/03	"	209.8	"	"	"	CF	HTEXCH	Nuclear Core Physics Test
83-054	7/12	"	200.2	"	"	"	CB	PUMPXX	Nuclear Reactor Coolant/ Recirculating Pumps
83-056	7/21	"	194.9	"	"	"	RC	VESSEL	Nuclear Core Physics Test
83-058	7/29	"	11.8	"	"	"	"	"	" " "
83-060	7/30	"	22.7	"	3	"	HA	TURBIN	Nuclear Steam Generator Controls

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4 Exhibit G Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1 Salem as Source

MAJOR PLANT MODIFICATIONS

REPORT MONTH JULY 1983

DOCKET NO: 50-311

UNIT NAME: Salem 2

DATE: August 10, 1983

COMPLETED BY: L. K. Miller

TELEPHONE: (609)935-6000 Ext. 4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2EC-0505	Fan Coil Units	Replace all coils of the fan coil units.
2EC-0665	Emergency Control Air Compressor	Provide emergency air compressor cooling from safety related chilled water system and disconnect existing service water system cooling.
2EC-0677	Switchgear and Penetration Areas	Revise dampers CAV12, CAV13 and CAV33 to fail open as per CD-M-80.
2EC-0761	Reactor Coolant System	Install a device to monitor the reactor vessel level.
2EC-0885	Steam Generator Drains & Blowdown	Reroute steam generator blowdown tank vent to No. 22 condenser shell.
2EC-1014	Component Cooling Water	Provide motor operators for component cooling water pumps discharge header valves No. 21-CC-3 and 22-CC-3.
2EC-1015	Component Cooling Water	Provide motor operators for component cooling pump suction header valves No. 2CC17 and 2CC18.
2EC-1056	Pressure and Delta P Transmitters	Replace pressure and differential pressure transmitters which must survive MSLB and LOCA conditions with transmitters qualified to IEEE323-1974.
2EC-1058	Hydrogen Recombiners	Modify design and change associated hardware.
2EC-1060	Auxiliary Feedwater	Replacement of the auxiliary flow transmitters with safety grade instruments.
2EC-1068	Radiation Monitoring System	Relocate RMS channels 2R19, 34 and 13 electronic equipment to an area of less temperature and humidity.
2EC-1185	Radiation Monitoring System	Addition of five high range main steam line noble gas monitors with detection capability of 10^3 μCi/cc. The monitors will be safety grade and qualified for post-accident operation.

* DESIGN CHANGE REQUEST
8-1-7.R1

MAJOR PLANT MODIFICATIONS

REPORT MONTH JULY 1983

DOCKET NO: 50-311

UNIT NAME: Salem 2

DATE: August 10, 1983

COMPLETED BY: L. K. Miller

TELEPHONE: (609)935-6000 Ext. 4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2EC-1216	Reactor Coolant Head Vent	Modify reactor head vent piping in the vicinity of the reactor head.
2EC-1253	Service Water	This change calls for the replacement of the two 16" return headers which are located below the valves SW223.
2EC-1254	Service Water	Provide interconnection for the service water headers (supply and return) for the chiller condenser and emergency air compressor areas.
2EC-1255	Auxiliary Feedwater	Install eight (8) check valves in auxiliary feedwater pump discharge piping at proposed locations.
2EC-1301	CVCS	Remove safety injection signal from valves 2CV139 and 2CV140; keeping valves open.
2EC-1315	Control Room Console	Provide cutouts for various design modifications.
2EC-1317	Control Room Side Panel Arrangement	Provide cutouts for the addition of instruments.
2EC-1332	RHR-Containment	Lower the existing anti-vortex baffle from El. 81'5 1/2 (bottom of grid) as noted in the Alden Research Lab Report.
2EC-1356	Diesel System	Provide a permanent source of DC power for alternate shutdown procedures for starting the diesel generators.
2EC-1363	Auxiliary Feedwater	Provide a 3" cross connection with two inline valves between the discharge lines on the two motor driven AF pumps.
2EC-1376	Feedwater	Replace BF22 feedwater stop check valves with power operated slow closure stop check valves or add such valves in series after disabling the BS22 check function.
2EC-1410	115 Volt AC E-153	Modify the 230V AC power feeds to the vital instrument inverters and the essential controls inverters.

* DESIGN CHANGE REQUEST

8-1-7.R1

MAJOR PLANT MODIFICATIONS

REPORT MONTH JULY 1983

DOCKET NO: 50-311

UNIT NAME: Salem 2

DATE: August 10, 1983

COMPLETED BY: L.K. Miller

TELEPHONE: (609)935-6000 Ext. 4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2EC-1414	Reactor Coolant	Replace wide range RTD's with environmentally qualified elements.
2EC-1452	Circulating Water	Replace 90/10 copper-nickel condenser tubes in Salem 2 main surface condenser - replacements as outlined in Detail Specification 81-6412.
2EC-1460	Fire Protection	Reroute fire protection piping in TGA discharge side of condenser to allow system to remain operative during condenser retubing.
2EC-1469	Platform Steel - TGA	Change weld supports to a bolted design on the platform steel access platform for the main steam dump valves to facilitate removal for the main condenser retubing.
2EC-1482	Turbine Bypass Steam	Increase the time suppression setting of the steam generator feed pump suction controls to 5.0 seconds.
2EC-1528	Circulating Water System Main Condenser Retubing	Remove interferences to enable retubing of the Unit No. 2 main condenser.
2EC-1543	125V DC Distribution (E220) Protective Relays (E560)	Install General Electric type 0275 A 4308 Lo-Pass Filter in the 125V DC line near the 4kV bus under frequency sensing relays.
2EC-1579	Component Cooling/Service Water	Retube No. 22 Component Cooling Heat Exchanger with titanium tubes in lieu of existing 90/10 CuNi tubes.
2EC-1580	Safety Injection System	Install the repaired 1SJ4 valve in the place of the removed 22SJ40 valve.
2EC-1603	Control Room Console	Replace the General Electric reactor manual trip switches on console. The new switches (2) will not have the removable handle option.
2EC-1612	Safety Injection System	Change valve packing for valves 2SJ4, 2SJ5, 2SJ12 and 2SJ13 (Valve Mark No. FA-76) from existing "John Crane 187-I" packing to new "Grafoil Halogenated Die Molded Split Ring Sets".

* DESIGN CHANGE REQUEST
8-1-7.R1

MAJOR PLANT MODIFICATIONS

REPORT MONTH JULY 1983

DOCKET NO: 50-311

UNIT NAME: SALEM 2

DATE: August 10, 1983

COMPLETED BY: L.K. Miller

TELEPHONE: (609)935-6000 Ext. 4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2EC-1621	Component Cooling	Provide for addition of handhole to No. 22 Component Cooling Heat Exchanger and the subsequent patching of same.
2EC-1636	Chemical and Volume Control	Replace valve 2CV81 and 2CV82, Mark No. FA-4 with valves having a similar design.
2SC-0274	Incore Thermocouple	Change the method of connecting field cable to the Incore Thermocouples.
2SC-0348	Component Cooling	Provide motor operated valve for the Auxiliary Header Isolation Valves 2CC30 and 2CC31.
2SC-0505	Main Turbine Lube Oil	Replace 21 & 22 main turbine lube oil cooler tubing bundles with alternate material or change cooler design to plate type exchangers.
2SC-0600	Structural	Install a cage on the ladder to #2 pressurizer.
2SC-0623	Service Water	Install a drainage system from the 10" inlet and outlet pipes of the containment fan coil units to the 20" service water discharge piping. System to consist of piping, valves and pumping mechanism to ensure that one FCU can be isolated and drained.
2SC-0651	Steam Generator Feedwater #21 and 22 Pumps - Pump Journal Bearings	Take new bearing retainers and modify them per Franklin Institute Research Lab Report F-A5477.
2SC-0744	Service Water	Install 316 SS piping, fittings and tubing in lube oil cooling system on both #21 and 22 Charging Pumps.
2SC-0746	Circulating Water	Install 316 SS fittings, valves and piping on the bearing lube piping to the circulator motors. Install break flanges in areas where carbon steel and stainless steel meet.

* DESIGN CHANGE REQUEST
8-1-7.R1

MAJOR PLANT MODIFICATIONS

REPORT MONTH JULY 1983

DOCKET NO: 50-311

UNIT NAME: SALEM 2

DATE: August 10, 1983

COMPLETED BY: L.K. Miller

TELEPHONE: (609)935-6000 Ext 4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2SC-0783	Containment Penetration Cooling	Cut penetration cooling air lines and install unions so that check valves can be removed and capped off to provide a positive test boundry.
2SC-0786	Service Water	Install 6" drain connection between valves 21SW23 and 22SW23.
2SC-1125	Containment Spray/Refueling Cavity	Provide a hard pipe for the containment spray cavity fill line.

* DESIGN CHANGE REQUEST
8-1-7.R1

MAJOR PLANT MODIFICATIONS
REPORT MONTH JULY

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: August 10, 1983
COMPLETED BY: L.K. Miller
TELEPHONE: 609/935-6000 X4455

*DCR NO.	10CFR50.59	SAFETY EVALUATION
2EC-0505		The installation of coils made of AL-6X tubing (instead of 90/10 Cu/Ni) and utilizing a water box header of (316 S.S.), both materials being more suitable for the prevailing service water conditions, does not affect any presently performed safety analysis, nor does it create any new safety hazards. No unreviewed safety or environmental questions are involved.
2EC-0665		Changing the cooling medium for the emergency air compressors from service water to chilled water does not affect any presently performed safety analysis, nor does it create any new safety hazards. No unreviewed safety or environmental questions are involved.
2EC-0677		The damper positions were not used in calculations for the presently performed safety analysis. No unreviewed safety or environmental questions are involved.
2EC-0761		Provision of local reading capability for the reactor vessel liquid level does not create any new safety hazards nor does it affect any other safety related functions. No unreviewed safety or environmental questions are involved.
2EC-0885		Installation of this DCR will not affect any presently performed safety analysis nor create any new safety hazards. No unreviewed safety or environmental questions are involved.
2EC-1014		Changing the valve operators from manual to motor operated will not involve any changes to the FSAR, or Technical Specifications. No unreviewed safety or environmental questions are involved.
2EC-1015		Changing the valve operators from manual to motor operated will not involve any changes to the FSAR, or Technical Specifications. No unreviewed safety or environmental questions are involved.

MAJOR PLANT MODIFICATIONS
REPORT MONTH JULY

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: August 10, 1983
COMPLETED BY: L.K. Miller
TELEPHONE: 609/935-6000 X4455

*DCR NO.	10CFR50.59	SAFETY EVALUATION
2EC-1056	This DCR replaces pressure transmitters located in harsh environments with pressure transmitters which have been environmentally tested. This change should improve the reliability of the system. No unreviewed safety or environmental questions are invovled.	
2EC-1058	This change assures the operator of a continuous indication of hydrogen concentration in the containment as required by NUREG 0737. No unreviewed safety or environmental questions are involved	
2EC-1060	This change is for a direct replacement of transmitters. No unreviewed safety or environmental questions are involved.	
2EC-1068	The equipment will be reconnected as per original design. No unreviewed safety or environmental questions are involved.	
2EC-1185	Installation of this DCR does not affect any presently performed safety analysis. No unreviewed safety or environmental questions are involved.	
2EC-1216	Installation of this DCR does not create any new safety hazards. No unreviewed safety or environmental questions are involved.	
2EC-1253	The two lines are upgraded to improve service and durability. No unreviewed safety or environmental questions are involved.	
2EC-1254	Installation of this DCR does not create any new safety hazards. No unreviewed safety or environmental questions are involved.	
2EC-1255	Addition of these valves will enhance safe operation of the unit by simplifying operator required action. No unreviewed safety or environmental questions are involved.	
2EC-1301	Installation of this DCR creates no new safety hazards. No unreviewed safety or environmental questions are involved.	

MAJOR PLANT MODIFICATIONS
REPORT MONTH JULY

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: August 10, 1983
COMPLETED BY: L.K. Miller
TELEPHONE: 609/935-6000 X4455

*DCR NO.	10CFR50.59	SAFETY EVALUATION
2EC-1315		Installation of this DCR creates no new safety hazards. No unreviewed safety or environmental questions are involved.
2EC-1317		This design change does not affect the safe shutdown of the reactor. No unreviewed safety or environmental questions are involved.
2EC-1332		This change does not affect the basic design and function of the RHR sump. No unreviewed safety or environmental questions are involved.
2EC-1356		Providing a permanent source of D.C. starting power for the diesel generator does not affect any safety function or margin. No unreviewed safety or environmental questions are involved.
2EC-1363		Providing the cross connection will enhance system flexibility such that each motor driven auxiliary feedwater pump can feed to all four or any of the four steam generators. No unreviewed safety or environmental questions are involved.
2EC-1376		These operators will ensure positive closure of the stop valves to preclude a release of radioactive material to the outside environment. No unreviewed safety or environmental questions are involved.
2EC-1410		This design change enhances plant safety and availability by allowing automatic transfer of vital bus loads in the event of a vital bus inverter failure. No unreviewed safety or environmental questions are involved.
2EC-1414		This design change involves a direct replacement of existing equipment. The system function will not change. No unreviewed safety or environmental questions are involved.
2EC-1452		This design change does not affect any presently performed safety analysis nor does it create any new safety hazards. No unreviewed safety or environmental questions are involved.

MAJOR PLANT MODIFICATIONS
REPORT MONTH JULY

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: August 10, 1983
COMPLETED BY: L.K. Miller
TELEPHONE: 609/935-6000 X4455

*DCR NO.	10CFR50.59	SAFETY EVALUATION
2EC-1460		Rerouting of the fire protection piping does not affect the operability of the fire protection system. No unreviewed safety or environmental questions are involved.
2EC-1469		Installation of the DCR does not create any new safety hazards. No unreviewed safety or environmental questions are involved.
2EC-1482		This DCR alters a system which is not safety related. The basic function of the equipment is unchanged. No unreviewed safety or environmental questions are involved.
2EC-1528		This change to the service water bay level indication system and change to the 4" backwash of the feedwater system does not change any presently performed safety analysis. No unreviewed safety or environmental questions are involved.
2EC-1543		This change adds a low pass filter electrically upstream of the 4Kv group bus underfrequency relays. No unreviewed safety or environmental questions are involved.
2EC-1579		The installation of titanium tubes will not affect the Technical Specifications or result in an unreviewed safety question. The structural integrity of the tube and shell heat exchanger remains intact.
2EC-1580		This change replaced the valve in kind. No unreviewed safety or environmental questions are involved.
2EC-1603		The replacement of the reactor trip switch with one of the same kind, except for the removable handle, does not alter the function of the system. No unreviewed safety or environmental questions are involved.
2EC-1612		The packing replacement will actually enhance the reliability of the valve. No unreviewed safety or environmental questions are involved.
2EC-1621		The addition of a patch fitting as described in the DCR does not present an unreviewed safety hazard. No unreviewed safety or environmental questions are involved.

MAJOR PLANT MODIFICATIONS
REPORT MONTH JULY

DOCKET NO.: 50-311
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*DCR NO.	10CFR50.59	SAFETY EVALUATION

2EC-1636 The pressure rating of the new valves is equivalent to the system pressure rating. The valve material is acceptable for the CVC system. No unreviewed safety or environmental questions are involved.

2SC-0274 This change introduces a superior method of connecting cables, as well as cable that has been environmentally qualified. No unreviewed safety or environmental questions are involved.

2SC-0348 The operators being added allow the valves to close more rapidly to prevent draining of the system. No unreviewed safety or environmental questions are involved.

2SC-0505 The replacement tube bundles and floating heads will upgrade the system because the new material is more resistant to the service water environment.

2SC-0600 The installation of a safety cage around the ladder leading to the No. 2 Pressurizer does not create any new safety hazards. No unreviewed safety or environmental questions are involved.

2SC-0623 Installation of drains for the fan coil unit instruments does not create any new safety hazards. Elimination of drainage onto the floor will eliminate one potential source of contamination. No unreviewed safety or environmental questions are involved.

2SC-0651 This change does not affect any presently performed safety analysis nor does it create any new safety hazards. No unreviewed safety or environmental questions are involved.

2SC-0744 This change will upgrade the vendor supplied copper based material with small bore piping made of a more corrosion resistant material. No unreviewed safety or environmental questions are involved.

2SC-0746 Installation of this DCR will not create any new safety hazards. No unreviewed safety or environmental questions are involved.

MAJOR PLANT MODIFICATIONS
REPORT MONTH JULY

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: August 10, 1983
COMPLETED BY: L.K. Miller
TELEPHONE: 609/935-6000 X4455

*DCR NO.	10CFR50.59	SAFETY EVALUATION
2SC-0783		The installation of the piping unions will not affect the basic function of the system. No unreviewed safety or environmental questions are involved.
2SC-0786		The addition of a low point drain does not alter the basic function or design concept of the system in any way. No unreviewed safety or environmental questions are involved.
2SC-1125		Installation of this DCR does not affect any presently performed safety analysis, not does it create any new safety hazards. No unreviewed safety or environmental questions are involved.

P S E & G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT NO. 2

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
901581	MD	2	22 RC PUMP	
			FAILURE DESCRIPTION:	DISASSEMBLE, INSPECT, REPLACE, REASSEMBLE RCP SEALS. 820721
			CORRECTIVE ACTION:	INSTALLED NEW SEALS. 830415
923952	OD	2	CONT AIR LOCK EL. 100	
			FAILURE DESCRIPTION:	INNER DOOR DRIVE CHAIN IS BROKEN. 830513
			CORRECTIVE ACTION:	REMOVED ONE LINK AND ADJUSTED TURNBUCKLES. DOOR SAT. 830531
923957	OD	2	CONT PRESS RELIEF	
			FAILURE DESCRIPTION:	CANNOT SET DIVISION DAMPER OPEN. 830601
			CORRECTIVE ACTION:	INSTALLED MISSING JUMPER BETWEEN TERMINAL 2 AND TERMINAL 12. 830601
934355	OD	2	23 RCS LOOP FLOW CH3	
			FAILURE DESCRIPTION:	CHANNEL HAS NO ANALOG INDICATION AND RP-4 LOW FLOW LIGHT DID NOT CLEAR WHEN RCP WAS RUN. 830613.
			CORRECTIVE ACTION:	TRANSMITTER DEFECTIVE. REPLACED FORCE MOTOR, DISC AND MATCHING COIL. 830621
934413	OD	2	23 CHARGING PUMP	
			FAILURE DESCRIPTION:	5 GPM LEAK TO WASTE LIQUID SYSTEM. 830616
			CORRECTIVE ACTION:	REPACKED ALL 5 CYLINDERS. 830623

P S E & G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT NO. 2

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
934423	OD	2	VALVE OP 24SJ54	
			FAILURE DESCRIPTION:	OVERLOADS TRIP WHEN ATTEMPT MADE TO CLOSE. 830616
			CORRECTIVE ACTION:	FOUND CLOSING COIL UNABLE TO PULL IN. CONTACTS & CONTACTOR BINDING UP. REPLACED CONTACTOR. 830617
937627	OD	2	VALVE 24MS15	
			FAILURE DESCRIPTION:	VALVE LEAKS BY SLIGHTLY. 810708
			CORRECTIVE ACTION:	REPLACED INSERT, ROD ASSY, RING, HOLDER AND GUIDE. 830620
937628	OD	2	VALVE 21MS11	
			FAILURE DESCRIPTION:	VALVE LEAKS BY SLIGHTLY. 810708
			CORRECTIVE ACTION:	REPLACED INSERT. BALANCE OF INTERNALS ACCEPTABLE. 830620
938343	OD	2	23 AUX FEED PUMP	
			FAILURE DESCRIPTION:	TRIP COIL IN HSD PANEL 207 APPEARS TO BE BURNED UP. 830707
			CORRECTIVE ACTION:	REPLACED SOLENOID COIL AND TRIP COIL. VALVE OPERATES PROPERLY. 830708
938610	OD	2	VALVE OP 24MS167	
			FAILURE DESCRIPTION:	DO NOT RECEIVE CLOSED LIGHT. 830703
			CORRECTIVE ACTION:	BLED OIL AND RELEASED AIR. ADDED ABOUT 20 GAL OF OIL DURING PROCESS. 830704
938615	OD	2	VALVE 21MS167	
			FAILURE DESCRIPTION:	VALVE CANNOT BE CLOSED HYDRAULICALLY. 830703
			CORRECTIVE ACTION:	STROKED VALVE. ADDED 2 POINTS OF OIL. 830704

P S E & G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 2

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
924091	OT	2	VALVES 21, 22, 24BF22	
			FAILURE DESCRIPTION:	MOTORS DRAWING TOO MUCH CURRENT. REPLACE WITH UNIT 1 MOTORS. 830616
			CORRECTIVE ACTION:	CHANGED LIMITORQUES AS REQUIRED. 830623
924443	RE	2	INCORE FLUX MAP SYS	
			FAILURE DESCRIPTION:	REPAIR F DRIVE AND SPARE COMPARATOR. 830426
			CORRECTIVE ACTION:	REPLACED F AND E DRIVE COMPARATORS. 830603.

REFUELING INFORMATION

COMPLETED BY: L.K. Miller

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: August 10, 1983
TELEPHONE: 609/935-6000
EXTENSION: 4455

Month July 1983

1. Refueling information has changed from last month:
   YES ____ NO X

2. Scheduled date for next refueling: March 31, 1984

3. Scheduled date for restart following refueling: June 10, 1984

4. A) Will Technical Specification changes or other license amendments be required?
   YES ____ NO X
   NOT DETERMINED TO DATE 8/1/83

   B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
   YES ____ NO X
   If no, when is it scheduled? February 1984

5. Scheduled date(s) for submitting proposed licensing action:
   March 1984 (if required)

6. Important licensing considerations associated with refueling:
   NONE

7. Number of Fuel Assemblies:
   A) Incore 193
   B) In Spent Fuel Storage 72

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: March 2000

8-1-7.R4

SALEM UNIT 2

OPERATIONS SUMMARY REPORT

JULY 1983

Unit No. 2 began the month shutdown as preparations for mode change continued. The unit entered Mode 4 at 1813 hours on July 3rd, and Mode 3 at 0915 hours on July 6th. Attempts were made to correct the seal leakage on No. 24 Reactor Coolant Pump (RCP). However, investigation revealed that it was necessary to replace the seal package. The unit was cooled down and the RCP seal package was replaced. Repairs were also made to No. 23 Coolant Charging Pump relief valve. A unit startup was commenced and Unit No. 2 entered Mode 2 at 1019 hours on July 23rd. On July 27th, the unit tripped due to a technician error while performing a functional test on Intermediate Range Channel N-35. On July 28th, at 1409 hours, a startup of Unit No. 2 was again commenced. The unit entered Mode 1 at 1946 hours, July 28, 1983, and was synchronized at 0458 hours on July 29th. At 0525 hours on July 29th, the unit tripped on a Steam Generator Lo-Lo Level trip on No. 22 Steam Generator during transfer of the feed controls from manual to automatic. The main feedwater regulating valve failed to open as the bypass valve closed. The Unit was again brought critical on July 29th and power was increased until the reactor tripped again at 0245 hours, July 30, 1983. Power was being reduced in preparation for turbine overspeed testing when the turbine tripped on a high level in No. 22 Steam Generator. The feed regulating valves were in manual control when the steam generator level swelled to 67%, causing a turbine trip and reactor trip. Unit No. 2 was again brought critical on July 30, 1983, and entered Mode 1 at 2217 hours. The unit was synchronized at 0059 hours, July 31, 1983, and a power escalation was begun.

PSE&G

Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

August 10, 1983

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 2
DOCKET NO. 50-311

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of July 1983 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Summary of Safety Related Maintenance
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

LKM:sbh
cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosures
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