

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

N C B E P 2 2 0 0 - 0 0 0 0 0 0 - 0 0 0 3 4 1 1 1 1 4 5
9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

REPORT SOURCE	60	61	DOCKET NUMBER							68	EVENT DATE					74	REPORT DATE					80			
	L	6	0	5	0	-	0	3	2	4	7	0	7	0	7	8	3	8	0	8	0	3	8	3	9

0 2 During unit power operation performance of Primary Containment Volumetric Average

0 3 Temperature test, PT-16.2, revealed a primary containment average air temperature of

0 4 137.07 °F. On 7-13-83 two performances of this PT revealed temperatures of 135.1 °F

0 5 and 136.0 °F respectively. The Technical Specifications limit for primary containment

0 6 average air temperature is < 135 °F. These events did not affect the health and safety

of the public.

7	8	9	ADDRESS	DATA	STATUS	COMP	VALUE
0	8						80

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE						COMP. SUBCODE		VALVE SUBCODE			
0	9	S	B	X	Z	Z	Z	Z	Z	Z	Z	Z	Z				
7	8	9	10	11	12	13	14	15	16	17	18	19	20				
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.							
17	8	3	—	0	6	2	/	0	3	L	—	0					
21	22	23	24	25	26	27	28	29	30	31	32						
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
X	Z	B	—	0	0	0	0	Y	N	Z	9	9	9	9			
18	19	20	21	22	23	24	25	26	27	28	29	30	31	32			
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47			

1 0 These events are attributed to seasonal ambient temperatures and routine plant

1 1 evolutions which increase the inlet temperature of RBCCW to the drywell air coolers.

1 2 Heat loads on the RBCCW system and heat generation in the drywell were reduced to lower

1 3 the primary containment average air temperature to $< 135^{\circ}\text{F}$.

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8 9 FACILITY STATUS 1 5 E 28 10 11 12 13 % POWER 0 9 9 29 14 15 16 17 18 19 20 21 22 23 24 25 26 27 OTHER STATUS 30 NA 31 32 33 34 35 36 37 38 39 40 41 42 43 METHOD OF DISCOVERY 44 B 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 DISCOVERY DESCRIPTION 32 Periodic Test 80

ACTIVITY CONTENT
RELEASED OF RELEASE

1 6 2 33 34

AMOUNT OF ACTIVITY (35) NA

LOCATION OF RELEASE (36) NA

PERSONNEL EXPOSURES		TYPE		DESCRIPTION
NUMBER				
1	7	0	0	0
		(37)		(38)
				NA

PERSONNEL INJURIES		41
NUMBER	DESCRIPTION	
		NA

7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60						
<p>LOSS OF OR DAMAGE TO FACILITY (43)</p> <p>TYPE DESCRIPTION</p>																																																											

1 9 Z 42 NA

8 9 10
PUBLICATION
ISSUED DESCRIPTION (45)
2 0 N 44
8308120059 830803
PDR ADOCK 05000324
S PDR
NRC USE ONLY
68 69 80

PHONE: (919) 457-9521

LER ATTACHMENT - RO #2-83-62

Facility: BSEP Unit No. 2

Event Date: July 7, 1983

During unit power operation, performances of Primary Containment Volumetric Average Temperature Test (PT-16.2) on July 7 and 13, 1983, revealed primary containment average air temperatures of 137.07°F (on July 7, 1983), 135.10°F, and 136.00°F (both on July 13, 1983). These values are attributed to increases in the inlet temperature of RBCCW to the drywell air coolers which reduce the heat transfer capacity of RBCCW. The increased RBCCW temperatures resulted from the following:

1. Seasonal increases in the temperature of the Service Water System cooling water to the RBCCW System heat exchangers.
2. Increased heat load on the RBCCW System which occurs whenever the RWCU System effluent is diverted to the main condenser (RWCU System reject), thereby bypassing the RWCU System regenerative heat exchangers which cool the RWCU System process inlet flow prior to cooling by RBCCW in the RWCU System nonregenerative heat exchangers.

These events were resolved by reducing the heat load on the RBCCW System (reduced RWCU System reject) and by reducing the speed of, and therefore the heat output from, the reactor recirculation pumps' motors. This permitted greater heat transfer to occur between the RBCCW System and the drywell air coolers. The reduction in reactor recirculation pump speed resulted in a reactor power level of approximately 93 percent.



Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429

USNRC REGION 3
ATLANTA, GEORGIA

83 AUG 8 A9:55

August 4, 1983

FILE: B09-13510C
SERIAL: BSEP/83-2519

Mr. James P. O'Reilly, Administrator
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street N.W.
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-324
LICENSE NO. DPR-62
LICENSEE EVENT REPORT 2-83-62

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 2, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

RMP/jo/LETJ03

Enclosure

cc: Mr. R. C. DeYoung
NRC Document Control Desk

OFFICIAL COPY

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