

**LICENSEE EVENT REPORT**

CONTROL BLOCK: [ ][ ][ ][ ][ ](1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

0 1 M S G G S 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 56

CON'T

REPORT SOURCE L 6 0 5 0 0 0 4 1 6 7 0 7 2 2 8 3 8 0 8 0 5 8 3 9

DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | On July 22, 1983, while performing a remote operated valve lineup for  
0 3 | the Component Cooling Water System it was discovered that the inlet  
0 4 | valve (cooling supply) to the RWCU nonregenerative heat exchanger was  
0 5 | closed. Therefore, RWCU was not actually operable when relied upon for  
0 6 | an alternate method of decay heat removal in accordance with Action 1  
0 7 | of T.S.3.4.9.2. This is reported pursuant to T.S.6.9.1.12.b.

7 8 9

0 9  
7 8

SYSTEM CODE  
9 C B 11

CAUSE CODE  
11 A 12

CAUSE SUBCODE  
12 X 13

COMPONENT CODE  
13 Z Z Z Z Z Z 14

COMP. SUBCODE  
19 Z 15

VALVE SUBCODE  
20 Z 16

17 LER NO  
REPORT NUMBER

EVENT YEAR  
21 8 22 3

SEQUENTIAL REPORT NO.  
23 0 24 9 25 0

OCCURRENCE CODE  
27 1 28 1 29

REPORT TYPE  
30 T 31

REVISION NO.  
32 0

ACTION TAKEN  
33 X 34

FUTURE ACTION  
34 X 18 19

EFFECT ON PLANT  
35 Z 20

SHUTDOWN METHOD  
36 Z 21

HOURS  
37 0 38 0 39 0 40

ATTACHMENT SUBMITTED  
41 Y 23

NPRD-4 FORM SUB.  
42 N 24

PRIME COMP. SUPPLIER  
43 Z 25

COMPONENT MANUFACTURER  
44 Z 45 9 46 9 47 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The valve had isolated when a relay was removed during troubleshooting  
11 of the Division I LSSS. The relay was installed and the valve was  
12 opened after discovering that it had isolated. The investigation and  
13 determination of corrective actions have not been completed. An  
14 update report is expected to be submitted by September 12, 1983.

FACILITY STATUS (28) 1 5 6 7 8 9  
 % POWER (29) 0 0 0 10 11 12  
 OTHER STATUS (30) NA 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27  
 METHOD OF DISCOVERY (31) C 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59  
 DISCOVERY DESCRIPTION (32) System Lineup 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

ACTIVITY CONTENT  
RELEASED OF RELEASE

1	6	2	33	NA
7	8	9	10	11

AMOUNT OF ACTIVITY (35)

NA

LOCATION OF RELEASE (36)

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	37	2	38	NA	39

PERSONNEL INJURIES		NUMBER		DESCRIPTION	
1	2	3	4	5	6
1	0	0	0	40	NA

LOSS OF OR DAMAGE TO FACILITY (43)  
TYPE DESCRIPTION  
1 9 7 8 9 10  
Z 42 NA  
8308120038 830805

PUBLICITY  
 ISSUED DESCRIPTION (45)  
 2 0 N 44 NA  
 S PDR  
 NRC USE ONLY  
 68 69

NRC USE ONLY

NAME OF PREPARER

Ron Byrd

PHONE

1-800-637-9286

SUPPLEMENTARY INFORMATION TO  
LER 83-090/01 T-0

Mississippi Power & Light Company  
Grand Gulf Nuclear Station - Unit 1  
Docket No. 50-416

Technical Specification Involved: 3.4.9.2  
Reported Under Technical Specification: 6.9.1.12.b

Event Narrative:

On July 16, 1983, a Limiting Condition for Operation was entered pursuant to Technical Specification 3.4.9.2 when Shutdown Cooling Loop "A" was declared inoperable due to standby service water "A" loop being inoperable (see LER 83-083). The Reactor Water Cleanup System was acting as the alternate method of decay heat removal. Shutdown cooling loop "B" was operable.

However, on July 22, 1983, while performing a remote operated valve lineup for the Component Cooling Water System it was discovered that the inlet valve to the Reactor Water Cleanup (RWCU) non-regenerative heat exchanger was closed. Therefore, the RWCU system was not actually operable as an alternate method of decay heat removal.

Valve P42-F103 is the inlet cooling supply valve to the non-regenerative heat exchangers. The valve is air operated and fails closed on a loss of power. The valve is shed by the Load Shedding and Sequencing System upon a loss of offsite power.

A maintenance work authorization was issued on July 19, 1983, to remove certain relays to isolate the Load Shedding and Sequencing System for troubleshooting the occurrences on July 16 (LER 83-083). The work instructions included removal of relay XK58 which isolated valve P42-F103. The work instructions did not include installation of jumper or instructions to block the valve open. The relay was installed and the valve was opened after discovering it had closed (isolated).

The investigation and determination of corrective actions have not been completed. An update report is expected to be submitted by September 12, 1983.



# MISSISSIPPI POWER & LIGHT COMPANY

*Helping Build Mississippi*

P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

August 5, 1983

83 AUG 9 49:09

JAMES P. MCGAUGHY, JR.  
VICE PRESIDENT

Office of Inspection & Enforcement  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta St., N.W., Suite 2900  
Atlanta, Georgia 30303

Attention: Mr. J. P. O'Reilly, Regional Administrator

Dear Mr. O'Reilly:

SUBJECT: Grand Gulf Nuclear Station  
Unit 1  
Docket No. 50-416  
License No. NPF-13  
File 0260/L-835.0  
RWCU Not Operable When  
Required  
LER 83-090/01 T-0  
AECM-83/0443

On July 22, 1983, while performing a remote operated valve lineup for the Component Cooling Water System it was discovered that the inlet valve to the Reactor Water Cleanup non-regenerative heat exchanger was closed. Therefore, the Reactor Water Cleanup System was not actually operable when relied upon for an alternate method of decay heat removal in accordance with Action (1) of Technical Specification 3.4.9.2. This is reported pursuant to Technical Specification 6.9.1.12.b. Attached is LER 83-090/01 T-0 with Supplementary Information.

Yours truly,

JPM:sap  
Attachment

cc: Mr. J. B. Richard (w/a)  
Mr. R. B. McGehee (w/o)  
Mr. T. B. Conner (w/o)  
Mr. G. B. Taylor (w/o)

Mr. Richard C. DeYoung, Director (w/a)  
Office of Inspection & Enforcement  
U. S. Nuclear Regulatory Commission  
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