

3150-0011

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

0	1
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REPORT	L	6	0	5	0	0	0	2	6	0	7	0	7	0	4	8	3	8	0	8	0	2	8	3	9
SOURCE	65	61	DOCKET NUMBER					68	69	EVENT DATE					74	75	REPORT DATE					80			

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During unit 1 refueling outage and normal operation of units 2 and 3, an

0 3 | alarm came in for reactor building elevation 593 fire protection panel

0 4 | 2-25-311, zone 2B, heat detector TE-39-46B. This alarm could have masked

0 5 | signals from other detectors which are required to be operable by T.S.

0 6 | 3.11.C.1. There was no effect on public health and safety. There are

0 7 | no redundant systems. The system was returned to service in approximately

0 8 | 4.25 hours.

SYSTEM CODE		CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE				COMP. SUBCODE	VALVE SUBCODE				
0	9	A	X	A	I	N	S	T	R	U	E	Z	
7	8	9	10	11	12	13	14	15	16	17	18	19	
EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE				REPORT TYPE		REVISION NO.			
17	LER/RO REPORT NUMBER	8	3	03		6		03		L		0	
20	21	22	23	24	25	26	27	28	29	30	31	32	
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.	
A	18	Z	19	Z		21		0000		Y		23	
20	21	22	23	24	25	26	27	28	29	30	31	32	33
PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER											
L		25											
34	35	36	37										
K		120											
38	39	40	41										

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)	
1	0
1	1
1	2
1	3
1	4

FACILITY STATUS			% POWER			OTHER STATUS			METHOD OF DISCOVERY			DISCOVERY DESCRIPTION					
1	5	E	28	1	0	0	29	NA	A	31	Operator observation						
ACTIVITY CONTENT RELEASED OF RELEASE						AMOUNT OF ACTIVITY						LOCATION OF RELEASE					
1	6	Z	33	Z	34	NA			NA								
PERSONNEL EXPOSURES																	
NUMBER			TYPE			DESCRIPTION											
1	7	0	0	d	37	Z	38	NA									
PERSONNEL INJURIES																	
NUMBER			DESCRIPTION														
1	8	0	0	d	40	NA											
LOSS OF OR DAMAGE TO FACILITY																	
TYPE			DESCRIPTION														
1	9	Z	42	NA													
PUBLICITY ISSUED																	
DESCRIPTION			NA														
2	0	M	44	NA													
NRC USE ONLY																	

NAME OF PREPARER S. W. Solley

PHONE (205) 729-0891

8308110292 830802
PDR ADDCK 05000260
PDR

LER SUPPLEMENTAL INFORMATION

BFRO-50-260 / 83036 Technical Specification Involved 3.11.C.1

Reported Under Technical Specification 6.7.2.b.(2)* Date Due NRC 8/3/83

Event Narrative:

Unit 1 was in a refueling outage, unit 2 was operating normally at 100-percent power, and unit 3 was operating at 97-percent power. Only the unit 2 reactor building elevation 593 was affected by this event. At approximately 2135 hours on July 4, 1983, TE-39-46B heat detector brought in alarm signals on the local and control room panels. The alarm would not clear and could have masked signals from other detectors required to be operable by Technical Specification 3.11.C.1. A fire watch was established per T.S. 3.11.C.2. Troubleshooting revealed that the cable tray heat detector had apparently been stepped on and bent thus causing the alarm. There was no evidence of work in progress at the detector and this detector could have been bent for some time before it began alarming. The Kidde model 897096 heat detector was straightened and all alarms and indications cleared. Surveillance Instruction 4.11.C.1 and .5 for TE-39-46B were successfully completed. The system was returned to service in approximately 4.25 hours. There are no redundant systems. There was no effect on public health and safety. A memorandum will be written to section supervisors to instruct their personnel to use caution when working around cable trays and a precautionary statement to avoid stepping on cable trays will be inserted into Electrical Instructions pertaining to cable activities around cable trays. Estimated completion date is January 1, 1984.

* Previous Similar Events:

BFRO-50-259/81023
259/76010
50-296/80025
296/79004

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

TENNESSEE VALLEY AUTHORITY

USNRC REGION 1
CHATTANOOGA, TENNESSEE 37401
ATLANTA, GEORGIA

1750 Chestnut Street Tower II

83 AUG 5 P12:39

August 2, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

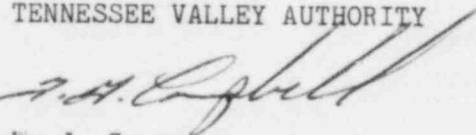
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET
NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE
REPORT BFRO-50-260/83036

The enclosed report provides details concerning a damaged heat detector
which initiated an alarm and thus could have masked other signals from
other detectors. This report is submitted in accordance with Browns
Ferry unit 2 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY


H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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Institute of Nuclear Power Operations
Suite 1500
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Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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