

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JUNE 1983

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle Nuclear Power Station Unit One is a Boiling Water Reactor with a designed electrical output of 1078 MWe net, located in Marseilles, Illinois. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. The plant is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit has not commenced commercial generation of power.

This report was compiled by Diane L. Lin, telephone number (815)357-6761, extension 499.

II. SUMMARY OF UNIT OPERATING EXPERIENCE FOR UNIT ONE

June 1 The unit started the reporting period at approximately 29% reactor power. At 0220 hours on June 1 the reactor scrammed due to low level while performing LOS-TG-SA1. The reactor was critical for 2 hours and 20 minutes.

June 2-6 The reactor went critical at 0930 hours on June 2. At 0055 hours on June 3 the main generator was synchronized to the grid and loaded. At 1811 hours on June 6 the reactor scrammed due to a turbine trip caused by high vibration. The reactor was critical for 104 hours and 41 minutes.

June 7-8 The reactor went critical at 2350 hours on June 7. At 0008 hours June 8 the reactor scrammed due to ranging from 6 to 7 during LGP-3-2. The reactor was critical for 18 minutes.

June 8-14 The reactor went critical at 0915 hours on June 8. At 0920 the main generator was synchronized to the grid and loaded to 20% power. On June 10 at 0800 reactor power was increased to 40%. On June 12 at 1225 reactor power was 69%. At 0719 hours on June 14 the reactor scrammed due to a turbine trip caused by high vibration on bearing #8. The reactor was critical for 142 hours and 4 minutes.

June 14-30 The reactor went critical at 2130 hours on June 14. At 0300 hours on June 15 the main generator was synchronized to the grid and loaded. At 0700 hours on June 15 reactor power was increased to 42%. At 1835 on June 15 reactor power was increased to 71%. On June 16 at 0347 load was reduced to 52% per the load dispatcher. At 0820 on June 16 reactor power was 67%. At 0800 on June 17 reactor power was increased to 62%. On June 26 at 0117 power was reduced to 40% for a rod sequence change. At 0135 on June 25 reactor power was 64%. On June 27 at 2350 reactor power was reduced to 30% for Reactor Recirculation testing. At 1530 on June 28 reactor power was 65%. At 2221 hours on June 30 the reactor scrammed due to startup testing, STP-27. The reactor was critical for 384 hours and 51 minutes.

SUMMARY OF UNIT OPERATING EXPERIENCE FOR UNIT ONE (Cont'd.)

0347 load was reduced to 52% per the load dispatcher. At 0820 on June 16 reactor power was 67%. At 0800 on June 17 reactor power was increased to 62%. On June 26 at 0117 power was reduced to 40% for a rod sequence change. At 0135 on June 25 reactor power was 64%. On June 27 at 2350 reactor power was reduced to 30% for Reactor Recirculation testing. At 1530 on June 28 reactor power was 65%. At 2221 hours on June 30 the reactor was scrammed as per startup test STP-27. The reactor was critical for 384 hours and 51 minutes.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications.

There were no amendments to the facility license or technical specifications during the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

C. Tests and Experiments Requiring NRC Approval.

There were no tests or experiments requiring NRC approval during the reporting period.

D. Corrective Maintenance of Safety Related Equipment.

The following tables present a summary of safety-related maintenance completed on Unit One during the reported period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Component Name, Cause of Malfunctions, Results and Effects on Safe Operation, and Corrective Action.

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
19883	---	LPRM 08-41A	Failed Downscale	None	Replaced connector
24206	---	Startrec Isolators	Out of calibration	None	Recalibrated points
24623	83-04-8/03L-0	Mechanical Snubber	Bad Snubber	Locked in place	Snubber replaced
24705	83-051/03L-0	B VC Chlorine Detector	Loose circuit board	Trip will not reset	Tightened circuit board.
24864	---	RHR IC System Pressure Gage	Miscalibration	Incorrect pressure readings	Calibrated Gage
24866	---	VG Effluent Radiation Monitor	Bad chart drive motor	No continuous recordings	Replaced chart drive motor
24947	83-055/03L-0	MS Leakage Control Temperature	None	Required for startup	Lifted leads and verified upscale indication
25046	---	SBGT Radiation Monitor OPL58JA	Bad filter	Low Vacuum Pump Alarms, Clears and repeats cycl.	Filter changed
25081	---	Narrow Range Level Transmitter & Level Indicator	Out of Calibration	Level Indicator does not track in the band	Calibrated logs
25145	---	OPL58JA SBG7 Radiation Monitor	Bad chart drive motor, bad vacuum transducer.	Monitor cycles from operate to fail mode	Replaced chart drive motor and vacuum transducer

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
25158	83-060/01L-0	RR Pump Breaker 4B	Unknown	None, since redundant 3B breaker	Troubleshooting performed logic test performed. No problem discovered.
25215	---	Div II Post Loca H ₂ /O ₂	Defective regulator, bad pump diaphragm	Div I available for post Loca Monitoring	Changed Regulator "RO" installed new diaphragms in pump new flow control valves or reagent gases
25250	---	U-1 HPCS AC Air Loading Relief Valve	Bad head gasket lodged over discharge valve	Over pressure resulted and relief valve lifted	Replaced head and spacer gaskets
25324	---	Div I Post Loca H ₂ Recorder/Monitor	Bad H ₂ + O ₂ reagent Gas Flowmeter Control Valves	Drifts Upscale	Changed H ₂ + O ₂ Reagent gas flow- meter control valves
25382	---	Div I Post Loca O ₂	Leak in 'O' ring on O ₂ cell.	Drifts Downscale	Replaced pump diaphragm, Replaced 'O' ring

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, June 1 to June 30, 1983. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
83-047/03L-0	5-07-83	Radwaste Discharge Process Radiation Monitor
83-048/03L-0	5-10-83	Mechanical Snubber Inoperable
83-049/03L-0	5-18-83	Mechanical Snubbers not deleted from Technical Specifications
83-051/03L-0	5-15-83	B VC chlorine Detector
83-052/03L-0	5-16-83	1B21-FD28C MSIV RPS Limit switch out of Tolerance
83-053/03L-0	5-22-83	Radwaste Discharge with inoperable Process Radiation Monitor
83-054/03L-0	5-20-83	Missed monthly functional or suppression Pool Water Temperature
83-055/03L-0	5-25-83	MS Leakage control Temperature Indication Functional
83-056/03L-0	5-29-83	Uncoupled Rod 30-19
83-057/03L-0	6-01-83	Coldown Rate Exceeded
83-058/01T-0	6-21-83	Primary Containment Vacuum Breaker Closed
83-059/01L-0	5-31-83	Failure of Radwaste Discharge Valve to Close
83-060/01L-0	6-06-83	RR Breaker 4B did not open on RPT

IV. UNIQUE REPORTING REQUIREMENTS

(Con't.)

B. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period:

<u>Outage No.</u>	<u>Equipment</u>	<u>Purpose of Outage</u>
1-605-83	"A" RHR Pump	Lube motor
1-608-83	"C" RHR Pump 1E12-C002C	Change oil
1-647-83	HPCS Pump	Change motor bearing oil
1-668-83	HPCS Minimum Flow Valve 1E22-F012	Rewire digital computer point
1-671-83	HPCS Switchgear 143	Breaker inspection
1-672-83	HPCS Diesel Generators	Replace contacts on relay K-18

C. Off-Site Dose Calculation Manual

There were no changes to the Off-Site Dose Calculations Manual during this reporting period.

D. Radioactive Waste Treatment System

There were no changes to the Radioactive Waste Treatment System during this reporting period.

E. Process Control Program

There were no changes to the Process Control Program during this reporting period.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LaSalle One
 DATE July 12, 1983
 COMPLETED BY Diane L. Lin
 TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD: June 1983 GROSS HOURS IN REPORTING PERIOD: 720
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 100% MAX. DEPEND. CAPACITY (MW_e-Net): 0
 DESIGN ELECTRICAL RATING (MW_e-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MW_e-Net): _____
 4. REASONS FOR RESTRICTION (IF ANY): _____

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>634.2</u>	<u>1791.2</u>	<u>5428.6</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0</u>	<u>0</u>	<u>0</u>
7. HOURS GENERATOR ON LINE	<u>588.9</u>	<u>1616.38</u>	<u>3474.08</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0</u>	<u>0</u>	<u>0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>1150742.0</u>	<u>2407383.0</u>	<u>4547962.0</u>
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	<u>352622</u>	<u>701930</u>	<u>1222329</u>
11. NET ELECTRICAL ENERGY GENERATED (MWH)	<u>331575</u>	<u>626241</u>	<u>1087016</u>
12. REACTOR SERVICE FACTOR	<u>NA</u>	<u>NA</u>	<u>NA</u>
13. REACTOR AVAILABILITY FACTOR	<u>NA</u>	<u>NA</u>	<u>NA</u>
14. UNIT SERVICE FACTOR	<u>NA</u>	<u>NA</u>	<u>NA</u>
15. UNIT AVAILABILITY FACTOR	<u>NA</u>	<u>NA</u>	<u>NA</u>
16. UNIT CAPACITY FACTOR (Using MDC)	<u>NA</u>	<u>NA</u>	<u>NA</u>
17. UNIT CAPACITY FACTOR (Using Design MWe)	<u>NA</u>	<u>NA</u>	<u>NA</u>
18. UNIT FORCED OUTAGE RATE	<u>NA</u>	<u>NA</u>	<u>NA</u>

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 7/3/83

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	<u> </u>	<u>6/21/82</u>
INITIAL ELECTRICITY	<u> </u>	<u>9/4/82</u>
COMMERCIAL OPERATION	<u>7/83</u>	<u> </u>

ATTACHMENT A
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-373
UNIT LaSalle One
DATE July 12, 1983
COMPLETED BY Diane L. Lin
TELEPHONE (815) 357-6761

MONTH June 1983

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>0</u>
2.	<u>0</u>
3.	<u>328</u>
4.	<u>344</u>
5.	<u>389</u>
6.	<u>305</u>
7.	<u>0</u>
8.	<u>0</u>
9.	<u>74</u>
10.	<u>333</u>
11.	<u>572</u>
12.	<u>663</u>
13.	<u>733</u>
14.	<u>205</u>
15.	<u>403</u>
16.	<u>649</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>524</u>
18.	<u>526</u>
19.	<u>633</u>
20.	<u>754</u>
21.	<u>733</u>
22.	<u>723</u>
23.	<u>710</u>
24.	<u>516</u>
25.	<u>693</u>
26.	<u>701</u>
27.	<u>671</u>
28.	<u>568</u>
29.	<u>481</u>
30.	<u>645</u>
31.	<u>---</u>

ATTACHMENT B
UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE July 12, 1983
COMPLETED BY Diane L. Lin.
TELEPHONE (815) 357-6761

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
16	6-01-83	F	31.2	A	3	Rx. scram due to low level while conducting LOS-TG-SA1
17	6-06-83	F	29.6	A	3	Rx. scram caused by a turbine trip on high vibration
18	6-08-83	F	9.1	A	3	Rx. scram due to ranging from 6 to 7 during LGP-3-2
19	6-14-83	F	14.2	A	3	Due to turbine trip from high vibration on breaking #8
20	6-16-83	F	0	H	5	Per load dispatcher
21	6-24-83	F	0	H	5	For sequence change
22	6-30-83	F	1.6	B	3	Turbine trip and Rx. scram for STP-27

VI. UNIQUE REPORTING REQUIREMENTS

A. Main Steam Relief Valve Operations for Unit 1

Relief valve operations during the reporting period are summarized in the following table. The table included information as to which relief valve was actuated, how it was activated and the circumstances resulting in its actuation.

<u>Date</u>	<u>Valves Actuated</u>	<u>No. & Type Actuations</u>	<u>Plant Conditions</u>	<u>Description of Events</u>
6-29-83	1B21-F013B	1 Manual	965 psig	STP-26
6-29-83	1B21-F013F	1 Manual	965 psig	STP-26
6-29-83	1B21-F013G	2 Manual	965 psig	STP-26
6-29-83	1B21-F013H	1 Manual	965 psig	STP-26
6-29-83	1B21-F013J	1 Manual	965 psig	STP-26
6-29-83	1B21-F013K	1 Manual	965 psig	STP-26
6-29-83	1B21-F013L	1 Manual	965 psig	STP-26
6-29-83	1B21-F013M	1 Manual	965 psig	STP-26
6-29-83	1B21-F013N	1 Manual	965 psig	STP-26
6-29-83	1B21-F013P	1 Manual	965 psig	STP-26
6-29-83	1B21-F013C	1 Manual	965 psig	STP-26
6-29-83	1B21-F013D	1 Manual	965 psig	STP-26
6-29-83	1B21-F013E	1 Manual	965 psig	STP-26
6-29-83	1B21-F013R	1 Manual	965 psig	STP-26
6-29-83	1B21-F013S	1 Manual	965 psig	STP-26
6-29-83	1B21-F013U	1 Manual	965 psig	STP-26
6-29-83	1B21-F013V	1 Manual	965 psig	STP-26
6-29-83	1B21-F013A	1 Manual	965 psig	STP-26



Commonwealth Edison
LaSalle County Nuclear Station
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Telephone 815/357-6761

July 12, 1983

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station, Unit One, for the period covering
June 1 through June 30, 1983.

Very truly yours,

CE Sargent

G. J. Diederich
Station Superintendent

GJD/DLL/bej

Enclosure

xc: J.G. Keppler NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright III. Dept. of Nuclear Safety
D.P. Galle CECO
D.L. Farrar CECO
INPO Records Center

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