

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-387

UNIT One

DATE July 11, 1983

COMPLETED BY L.A. Kuzcynski

TELEPHONE (717) 542-2181

MONTH June, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>1020</u>
2	<u>1030</u>
3	<u>1046</u>
4	<u>1042</u>
5	<u>1044</u>
6	<u>1043</u>
7	<u>987</u>
* 8	<u>1048</u>
9	<u>1053</u>
10	<u>1050</u>
11	<u>1045</u>
12	<u>1041</u>
13	<u>1039</u>
14	<u>414</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>638</u>
18	<u>896</u>
19	<u>903</u>
20	<u>1017</u>
21	<u>1036</u>
22	<u>1042</u>
23	<u>1041</u>
24	<u>484</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u></u>

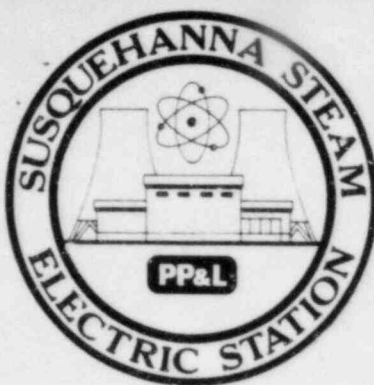
## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

\* Unit was declared commercial at 0001 on June 8, 1983.

(9/77)

8308110143 830711  
PDR ADOCK 05000387  
R PDR



# OPERATING DATA REPORT

DOCKET NO. 50-387  
 DATE July 11, 1983  
 COMPLETED BY L.A. Kuczynski  
 TELEPHONE (717)542-2181

## OPERATING STATUS

Unit 1

1. Unit Name: Susquehanna Steam Electric Station
2. Reporting Period: June, 1983
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1072
7. Maximum Dependable Capacity (Net MWe): 1036

### Notes

\* Unit was declared commercial at 0001 on June 8, 1983

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7, Since Last Report, Give Reasons:  
Items 6 and 7 will vary month-to-month until sufficient operating data has been gathered.

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	552	552	552
12. Number Of Hours Reactor Was Critical	351.6	351.6	351.6
13. Reactor Reserve Shutdown Hours	156.7	156.7	156.7
14. Hours Generator On-Line	334.3	334.3	334.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,050,000	1,050,000	1,050,000
17. Gross Electrical Energy Generated (MWH)	341,760	341,760	341,760
18. Net Electrical Energy Generated (MWH)	329,910	329,910	329,910
19. Unit Service Factor	60.6	60.6	60.6
20. Unit Availability Factor	60.6	60.6	60.6
21. Unit Capacity Factor (Using MDC Net)	57.7	57.7	57.7
22. Unit Capacity Factor (Using DER Net)	56.1	56.1	56.1
23. Unit Forced Outage Rate	39.4	39.4	39.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):  
Unit 1/Unit 2 Tie-in Outage to Commence November 7, 1983. Scheduled to last 29 days.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: 3 July 1983

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____



# UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June, 1983

DOCKET NO. 50-387  
 UNIT NAME One  
 DATE July 11, 1983  
 COMPLETED BY L.A. Kuczynski  
 TELEPHONE (717) 542-2181

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
12	061483	F	61	A	3	N/A	CC	VALVEX	Scram caused by a momentary three times normal main steam line radiation level. High rad. level caused by an air slug passed through the core. The air slug originated from a condensate demineralizer which had its vent line blocked by some gasket-type material which prevented proper venting. Investigation into action to prevent recurrence is still under investigation.
13	062483	F	156.7	A	3	83-092/03L	EA	TRANSF	A startup transformer failed in service. This caused a reactor scram and main turbine trip on RPV high level. Present replacement transformer will be replaced with transformer having automatic load tap changing.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

SUSQUEHANNA STEAM ELECTRIC STATION

Docket Number 50-387 Date July 11, 1983

Completed by L.A. Kuczynski Telephone (717)542-2181

Challenges to Main Steam Safety Relief Valves

Five minutes after the scram on June 15, 1983, reactor pressure reached 1080 psig and safety relief valve "E" automatically opened to relieve the pressure. It reseated properly.

Changes to the Offsite Dose Calculation Manual

See attachment.

Major Changes to Radioactive Water Treatment Systems

None.

ATTACHMENT TO  
June, 1983, Monthly Operating Report

Changes to the  
Offsite Dose Calculation Manual

These revised pages were made effective on April 27, 1983 upon signature by the Manager-Nuclear Support.

Changes have been denoted by revision bars in the right margin. The reasons for the changes are as follows:

- ° To reflect the current text of the applicable Technical Specifications where they are quoted (Pages 7, 8, 9, 14, 15, 16, 21, 25, 26, 30, 32, 34, 39a, 40, 48).
- ° To clarify the fact that dose rate calculations are performed monthly as well as "when necessary" (vent monitor alarm) (Page 3).
- ° To include applicable 10CFR20 dose rate standards in the table of radiological effluent objectives and standards (Page 6).
- ° To clarify the fact that Table 3 meteorological data is for sample purposes only. For dose calculations for the Semi-annual Radioactive Effluent Release Report, actual meteorological data from the report period is used (Pages 17, 18, 19, 25, 27).
- ° To change the dose parameters for radionuclides other than noble gases via the ground plane pathway. The former values, which were based on total body dose factors for standing on contaminated ground, were less conservative than the current values, which are based on skin dose from standing on contaminated ground (Page 20).
- ° To include the statement that, when calculating doses to gauge compliance with Specification 3.11.2.5, the ventilation exhaust treatment system operability specification, only doses from I-131, tritium, and particulates with half-lives greater than eight days shall be included. This is because, by definition in Tech. Spec. Definition 1.47, the ventilation exhaust treatment system is not considered to have any effect on noble gas effluents (Page 34).
- ° To add Appendix D, which presents site specific values used in the LADTAP II code to calculate doses from liquid effluents (Pages D-1, D-2, D-3).





# Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

Bruce D. Kenyon  
Vice President-Nuclear Operations  
215/770-7502

July 14, 1983

Director, Data Automation &  
Management Information Division  
Attention: Mr. M. R. Beebe  
Management Information Branch  
Office of Resource Management  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

SUSQUEHANNA STEAM ELECTRIC STATION  
MONTHLY OPERATING REPORT - JUNE 1983  
ER 100450 FILE 841  
PLA-1745

Docket No. 50-387

Dear Mr. Beebe:

The June 1983 monthly operating report for Susquehanna SES Unit 1 is attached.  
The Unit was declared commercial at 0001 on June 8, 1983.

Very truly yours,

for B. D. Kenyon  
Vice President-Nuclear Operations

Attachment

cc: Dr. Thomas E. Murley  
Regional Administrator-Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

INPO Records Center  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, Georgia 30339

Director  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555  
Attn: Document Control Desk (12 copies)

Mr. Thomas E. Pollog  
Department of Environmental  
Resources  
Bureau of Radiation Protection  
P. O. Box 2063  
Harrisburg, PA 17120

Mr. G. Rhoads - NRC  
Mr. R. Perch - NRC

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