

## LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 N Y N M P 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
7 8 9 LICENSE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T

0 1 REPORT SOURCE L 6 0 5 0 0 0 2 2 0 7 0 7 1 1 8 3 8 0 7 2 5 8 3 9  
7 8 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2  
0 3 SEE ATTACHED SHEET.  
0 4  
0 5  
0 6  
0 7  
0 8

0 9 SYSTEM CODE S B 11 CAUSE CODE B 12 CAUSE SUBCODE A 13 COMPONENT CODE V A L V O P 14 COMP SUBCODE D 15 VALVE SUBCODE Z 16  
7 8 9 10 11 12 13 18 19 20  
17 LER RO REPORT NUMBER 8 3 21 24 23 24 26 27 28 29 30 31 32 REVISION NO. 0  
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS 22 ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER  
X 18 A 19 Z 20 Z 21 0 0 0 0 0 Y 23 N 24 L 25 H 1 9 8 26  
33 34 35 36 37 40 41 42 43 44 47  
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0  
1 1 SEE ATTACHED SHEET.  
1 2  
1 3  
1 4

1 5 FACILITY STATUS E 28 1 0 0 29 NA 30 METHOD OF DISCOVERY C 31 NRC Inspection of Shielding Review 32  
7 8 9 10 12 13 44 45 46 80  
1 6 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 Z 34 NA 35 AMOUNT OF ACTIVITY LOCATION OF RELEASE 36  
7 8 9 10 11 44 45 80  
1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 Z 38 NA 39 DESCRIPTION  
7 8 9 11 12 13 80  
1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 NA 41 DESCRIPTION  
7 8 9 11 12 80  
1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 NA 43 DESCRIPTION  
7 8 9 10 80  
2 0 PUBLICITY ISSUED N 44 NA 45 DESCRIPTION  
7 8 9 10 80  
8308090688 830725  
PDR ADOCK 05000220  
S PDR  
NRC USE ONLY  
7 8 9 10 80

NAME OF PREPARER Richard C. Neild

PHONE (315) 349-2467

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

An NRC inspection of NUREG 0737, shielding review, revealed that the four containment spray test valves (EP numbers 80-40, 80-41, 80-44 and 80-45) may not meet the seismic criteria due to the installation of air operators. These operators were installed without proper seismic analysis.

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

The cause of this problem was the failure to perform a seismic analysis in the design of the modification. On July 11, 1983, the air operators on valves 80-44 and 80-45 were removed to restore one containment spray system to an acceptable configuration for seismic operability of one system. The following day, July 12, 1983, the air operators on valves 80-40 and 80-41 were then removed to place both containment spray systems in an acceptable condition for seismic operability and to comply with Technical Specification 3.3.7. A seismic analysis of the containment spray test/alternate torus cooling lines affected by the installation of the subject valve operators is being performed to determine the seismic acceptability of these systems. The results of this analysis will follow as revision 1 to this LER. Using the results of the analysis, necessary modifications, if any, that are dictated by the results of the analysis will be implemented prior to reinstallation of the valve operators.

A review of all plant modifications completed from the commencement of the 1981 refueling outage to date which involved seismic considerations is underway to assure that the analysis methods used were correct and the modifications comply with our Final Safety Analysis Report.

July 25, 1983

Dr. Thomas E. Murley  
Regional Administrator  
United States Nuclear Regulatory Commission  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

RE: Docket No. 50-220  
LER 83-20

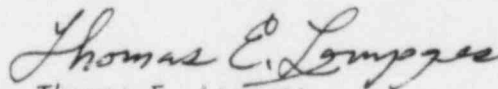
Dear Dr. Murley,

In accordance with Nine Mile Point Nuclear Station Unit #1 Technical Specifications, we hereby submit the following Licensee Event Report:

83-20      which is being submitted in accordance with section 6.9.2a(9), Performance of structures, systems or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses, in the safety analyses, in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

This report was completed in the format designated in NUREG-0161, dated July 1977.

Very truly yours



Thomas E. Lempges  
Vice President  
Nuclear Generation

TEL/jm  
Attachments (3 copies)  
cc: Director, Office of I&E (30 copies)  
Director, Office of MIPC (3 copies)

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