

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 0 H D B S 1 2 0 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

0 1 REPORT SOURCE L 6 0 5 0 0 0 3 4 6 7 1 0 2 6 8 1 8 0 7 1 2 8 3 9
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 (NP-33-81-82) On 10/26/81 at 1230 hours, control rod drive (CRD) trip breaker "B"
0 3 failed to trip consistently during the performance of PT 5155.02, "Anticipatory Reac-
0 4 tor Trip System Trip Test". The station entered the action statement of Technical
0 5 Specification 3.3.2.2, Table 3.3-1. There was no danger to the health and safety of
0 6 the public or station personnel. The redundant breaker and the redundant channel
0 7 were both operable throughout this occurrence.

0 8 7 8 9 80

0 9 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
I E 11 E 12 A 13 C K T B R K 14 A 15 Z 16
7 8 10 11 12 13 18 19 20
17 LER RO REPORT NUMBER 18 1 21 22 23 24 25 26 27 28 29 30 31 32
18 1 21 22 23 24 25 26 27 28 29 30 31 32
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
C 18 Z 19 Z 20 Z 21 0 0 0 22 Y 23 Y 24 N 25 G 0 8 0 26
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of the failure of the trip breaker was due to binding in the undervoltage
1 1 coil device linkage. The failed breaker was replaced with a spare. The new breaker
1 2 was satisfactorily bench tested and installed. On 10/26/81 at 2010 hours, CRD trip
1 3 breaker "B" was declared operable. The CRD trip breakers have been incorporated in
1 4 the Preventive Maintenance Program.

1 5 7 8 9 80

1 5 FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)
E 28 0 9 7 29 NA B 31 During performance of PT 5155.02
7 8 9 10 11 12 13 44 45 46 80

1 6 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)
Z 33 Z 34 NA NA
7 8 9 10 11 44 45 80

1 7 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)
0 0 0 37 Z 38 NA
7 8 9 10 11 12 13 80

1 8 PERSONNEL INJURIES NUMBER DESCRIPTION (41)
0 0 0 40 NA
7 8 9 10 11 12 80

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)
Z 42 NA
7 8 9 10 80

2 0 PUBLICITY ISSUED DESCRIPTION (45)
N 44 NA
7 8 9 10 80

8308040439 830712
PDR ADOCK 05000346
S PDR

NRC USE ONLY

PHONE: 419-259-5000, Ext. 565

TOLEDO EDISON COMPANY
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE
SUPPLEMENTAL INFORMATION FOR LER NP-33-81-82

DATE OF EVENT: October 26, 1981

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Intermittent failure of Control Rod Drive (CRD) trip breaker "B"

Conditions Prior to Occurrence: The unit was in Mode 1, with Power (MWT) = 2696 and Load (Gross MWE) = 880.

Description of Occurrence: On October 26, 1981 at 1230 hours, CRD trip breaker "B" failed to trip consistently during the performance of Periodic Test PT 5155.02, "Anticipatory Reactor Trip System Trip Test". At that time, the station entered the action statement of Technical Specification 3.3.1.1, Table 3.3-1. The "B" breaker was placed in the tripped condition within one hour as required by the action statement.

1 | Designation of Apparent Cause of Occurrence: The cause of this occurrence was the failure of the trip breaker. The root cause has been identified to be due to binding in the undervoltage coil device linkage.

Analysis of Occurrence: There was no danger to the health and safety of the public or station personnel. The redundant breaker and the redundant channel were both operable throughout this occurrence.

1 | Corrective Action: The corrective action was to replace the breaker with an on-site spare. Under Maintenance Work Order 81-3704, the new breaker was satisfactorily bench tested and installed. Applicable portions of PT 5155.02 were successfully performed on the new breaker and at 2010 hours on October 26, 1981, CRD trip breaker "B" was declared operable and closed. Under Maintenance Work Order 81-3744, the failed breaker was disassembled, cleaned, lubricated, reassembled, and the undervoltage coil dropout adjusted. This work was completed July 7, 1982. The other CRD trip breakers were inspected by July 30, 1982 under Maintenance Work Order 82-1928, however, no similar problems were found. The CRD trip breakers have been incorporated in the Preventive Maintenance Program on a refueling frequency.

1 | At the time of this occurrence, periodic maintenance on these breakers was performed under PT 5105.02, "GE Type A.K. Circuit Breaker Periodic Test". All of the recommendations of GE Service Advice #179-9.3 had been incorporated into this procedure. PT 5105.02 has since been replaced by MP 1405.05, "480 Volt Breaker Maintenance".

The CRD trip breakers are now exercised monthly during the performance of ST 5030.12, "Channel Functional Test of the Reactor Trip Module Logic and CRD Trip Breakers", and ST 5030.19, "ARTS Monthly Functional Test". In addition, a trip response test of each CRD trip breaker is performed

TOLEDO EDISON COMPANY
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE
SUPPLEMENTAL INFORMATION FOR LER NP-33-81-82 PAGE 2

following the performance of MP 1405.05. This test involves connecting a variable voltage source to the UV coil input and a digital timer and chart recorder to the breaker main contacts. The time required to open the main contacts after the UV coil de-energizes is measured. The times measured to date have been in the 15-25 millisecond range with a maximum allowable limit of 50 milliseconds. ST 5030.20, "CRD Trip Breaker Response Testing" was written to perform this test.

Failure Data: A previous similar failure of a CRD trip breaker was reported in Licensee Event Report NP-33-80-100 (80-080).

LER #81-070



July 12, 1983

Log No. K83-1002

File: RR 2 (NP-33-81-82)

Docket No. 50-346

License No. NPF-3

Mr. James G. Keppler
Regional Administrator, Region III
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

Enclosed are three copies of Revision 1 to Licensee Event Report 81-070, including revised supplemental information sheets. The revisions to the report are indicated by a "1" in the left margin of each page. Please replace your previous copies of this report with the attached revision.

Yours truly,

Terry D. Murray
Station Superintendent
Davis-Besse Nuclear Power Station

TDM/ljk

Enclosure

cc: Mr. Richard DeYoung, Director
Office of Inspection and Enforcement
Encl: 30 copies

Mr. Norman Haller, Director
Office of Management and Program Analysis
Encl: 3 copies

Mr. Walt Rogers
NRC Resident Inspector
Encl: 1 copy

JUL 20 1983