

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/2/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/9/ (7) /0/7/0/6/8/3/ (8) /0/7/2/7/8/3/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On July 6, 1983, with Unit 2 at 100 percent power Fire Door S71-18 between "2H" /
/0/3/ / Emergency Diesel Generator Room and the Turbine Building would not latch and /
/0/4/ / lock. Since a fire watch was immediately posted the public health and safety /
/0/5/ / were not affected. This event is contrary to T.S. 3.7.15 and reportable pursuant /
/0/6/ / to T.S. 6.9.1.9.b. Degradation of fire doors between the Emergency Diesel Room /
/0/7/ / and the Turbine Building is a recurring event due to a differential pressure /
/0/8/ / during diesel operation. /

SYSTEM CAUSE CAUSE COMP. VALVE
CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE

/0/9/ /A/B/ (11) /E/ (12) /B/ (13) /X/X/X/X/X/X/ (14) /Z/ (15) /Z/ (16)
SEQUENTIAL OCCURRENCE REPORT REVISION
LER/RO EVENT YEAR REPORT NO. CODE TYPE NO.

(17) REPORT
NUMBER /8/3/ /-/ /0/5/5/ /-/ /0/3/ /L/ /-/ /0/

ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT
TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB. SUPPLIER MANUFACTURER

/A/ (18) /X/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /N/ (23) /N/ (24) /A/ (25) /C/1/7/5/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / Fire door S71-18 would not latch because of latch mechanism damage. The latching/
/1/1/ / mechanism was replaced and operability verified. Similar events were reported /
/1/2/ / in LER's 83-026, 034 and 045. Design Modifications to the fire doors between the/
/1/3/ / Emergency Diesel Room and Turbine Building are being pursued. /
/1/4/ /

FACILITY METHOD OF
STATUS %POWER OTHER STATUS DISCOVERY DISCOVERY DESCRIPTION (32)
/1/5/ /E/ (28) /1/0/0/ (29) / NA / (30) /A/ (31) / Operator Observation /

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)
/1/6/ /Z/ (33) /Z/ (34) / NA / / NA /

PERSONNEL EXPOSURES
NUMBER TYPE DESCRIPTION (39)
/1/7/ /0/0/0/ (37) /Z/ (38) / NA /

PERSONNEL INJURIES
NUMBER DESCRIPTION (41)
/1/8/ /0/0/0/ (40) / NA /

LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION
/1/9/ /Z/ (42) / NA /

PUBLICITY
ISSUED DESCRIPTION (45)
/2/0/ /N/ (44) / NA /

NRC USE ONLY

/ / / / / / / / / / / / / /

NAME OF PREPARER E. Wayne Harrell

PHONE (703) 894-5151

Vepco

USNRC REGION II
ATLANTA, GEORGIA

VIRGINIA ELECTRIC AND POWER COMPANY

83-084-1 A10-29
NORTH ANNA POWER STATION

P. O. BOX 402

MINERAL, VIRGINIA 23111

July 27, 1983

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

Serial No. N-83-106
NO/CLF: 11
Docket No. 56-339
License No. NPF-7

Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 2.

Report No.

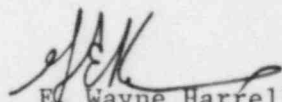
Applicable Technical Specifications

LER 83-055/03L-0

T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,


E. Wayne Harrell
for Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)
016 Phillips Bldg.
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

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