

## LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	S	C	H	B	R	2	0	0	-	0	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5		
7	8	LICENSEE CODE						14	LICENSE NUMBER										25	26	LICENSE TYPE					30	CAT 58				

CON'T

REPORT SOURCE: 01 L 6 0 5 0 0 0 2 6 1 7 0 6 2 5 8 3 8 0 7 2 5 8 3 9  
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | On June 25, 1983, at approximately 1120 hours, with the unit at 79% power, the

03 | Reactor Coolant System (RCS) leak rate was determined to be 1.07 GPM. This event

04 resulted in an unidentified source of RCS leakage and a total system leakage of

0	5	greater than one GPM as defined by Technical Specification 3.1.5.1 which is reportable
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06 | pursuant to 6.9.2.b.4. The installed plant safety systems were capable of performing

07 | their intended functions. There was no threat to the public health and safety.

7 8 9

0 9

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

C B 11 E 12 B 13 V A L V E X 14 E 15 D 16

9 10 11 12 13 14 15 16 17 18 19 20

(17) LER/RO REPORT NUMBER [ 8 | 3 ] [ — ] [ 0 | 1 | 3 ] [ / ] [ 0 | 3 ] [ L ] [ — ] [ 0 ]  
 21 22 23 24 25 26 27 28 29 30 31 32

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X	18 A	Z	Z	22	y	N	A	v o   8   5
33	34	35	36	37 40	41	42	43	44

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | Investigation revealed the source of leakage to be the packing of pressurizer

11 block valve RC-536. The block valve was placed on its backseat reducing the total

1 2 RCS leakage to within the limits allowed by Technical Specification 3.1.5.1. RC-536

13 will be repacked during the next cold shutdown of sufficient length to accomplish the

1	4
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 repairs. No further corrective action is considered necessary.

FACILITY STATUS			% POWER			OTHER STATUS			METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
1	5	E	0	7	9	N/A			A	Operator Observation		
7	8	9	10	11	12	13	14	15	16	17	18	

ACTIVITY CONTENT  
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35)  
1 6 Z (33) Z (34) N/A  
7 8 9 10 11 44

LOCATION OF RELEASE (36)  
N/A  
45 80

PERSONNEL EXPOSURES										
NUMBER		TYPE		DESCRIPTION						
1	7	0	0	0	(37)	Z	(38)	N/A		
7	8	9	11	12	13	80				

PERSONNEL INJURIES									
NUMBER			DESCRIPTION						
1	8	0	0	0	40	N/A			
7	8	9	11	12					

LOSS OF OR DAMAGE TO FACILITY					
TYPE		DESCRIPTION			
1	9	Z	(42)		N/A

7 8 9 10

8308010401 830725

ISSUED		DESCRIPTION		PDR		PDR		NRC USE ONLY	
2	0	N	44	PDR ADCK 05000261		PDR			
7	8	9	10						

NAME OF PREPARER Howard T. Cox

PHONE: (803) 383-4524

SUPPLEMENTAL INFORMATION  
FOR  
LICENSEE EVENT REPORT 83-013

I. Cause Description and Analysis

On June 25, 1983, at approximately 1120 hours, with the unit at 79% power, the Reactor Coolant System (RCS) leak rate was determined to be 1.07 GPM. This leak rate was determined by the performance of Periodic Test (PT) 8.0, RCS Leakage Evaluation.

This event resulted in an unidentified source of RCS leakage and a total system leakage of greater than one GPM as defined by Technical Specification 3.1.5.1 which is reportable pursuant to 6.9.2.b.4. The installed plant safety systems were capable of performing their intended function. There was no threat to the public health and safety.

II. Corrective Action

Investigation revealed increasing level in the Pressurizer Relief Tank (PRT) indicating that the leakage source was possibly from the pressurizer power operated relief block valves. Pressurizer block valve RC-536 was, therefore, opened and manually placed firmly on its backseat to determine if excessive packing leakage existed. PT-8.0 was again performed with a resulting leak rate of 0.31 GPM. In addition, the PRT level remained stable throughout the test. Thus it was determined that the leakage source was from the packing of RC-536 and the leakage was being directed via the leak-off line to the PRT. RC-536 was left backseated and continued operation in this configuration has been fully satisfactory since the time of the event. The backseated position of RC-536 does not affect the operability of the valve, and it can be closed should the need arise. The identification and correction of this leakage source terminated the limiting condition for operation of Technical Specification 3.1.5.1 and placed the total RCS leakage within the limit allowed by that specification.

III. Corrective Action to Prevent Recurrence

RC-536 will be repacked during the next cold shutdown of sufficient length to accomplish the repairs. No further corrective action is considered necessary.



Carolina Power & Light Company

USNRC REGION I  
ATLANTA, GEORGIA

H. B. ROBINSON STEAM ELECTRIC PLANT  
Post Office Box 790  
Hartsville, South Carolina 29550

83 JUL 28 49:29

JUL 25 1983

Robinson File No: 13510C

Serial: RSEP/83-980

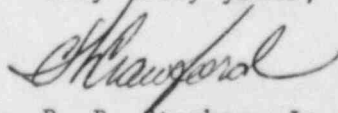
Mr. James P. O'Reilly  
Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 3100  
Atlanta, Georgia 30303

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261  
LICENSE NO. DPR-23  
LICENSEE EVENT REPORT 83-013

Dear Mr. O'Reilly:

In accordance with Section 6.9.2 of the Technical Specifications for the H. B. Robinson Steam Electric Plant, Unit 2, the enclosed Licensee Event Report is submitted. This report fulfills the requirements for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-0161, July, 1977.

Very truly yours,

  
for R. B. Starkey, Jr.  
General Manager

H. B. Robinson SEG Plant

HTC:FMG:FLL:CWC/th

Enclosure

cc: R. C. DeYoung (30)  
R. A. Hartfield (3)

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