NRC FORM 366 (7-77) Update Report-Previous Report Date 6-2-83

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | M S G G S 1 (2) 0 0 - 0 0 0 0 0 - 0 0 (3) 4 1 1 1 1 (4) | | (5)
LICENSEE CODE | LICENSE NUMBER | LICENSE TYPE | CAT

CON'T 0 1 REPORT SOURCE L (6) 0 5 0 0 0 4 1 6 (7) 0 4 1 1 8 3 (8) 0 7 1 8 8 3 (9)
DOCKET NUMBER | EVENT DATE | REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 On April 11, 1983, it was reported to the Shift Superintendent that the
0 3 vessel head/vessel flange thermocouples used for compliance with the
0 4 Technical Specification Surveillance Requirements of Technical
0 5 Specification 3.4.6.1, were not physically mounted on their respective
0 6 surfaces. The event was originally reported pursuant to Technical
0 7 Specification 6.9.1.13.b. The event had no affect on the health and
0 8 safety of the public and did not constitute a threat to plant safety.

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	C A (11)	D (12)	Z (13)	Z Z Z Z Z Z (14)	Z (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 5 2	/	0 3	X	—	2

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	Z (25)	Z 9 9 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 This event should not have been reported. See the attached
1 1 Supplementary Information to this LER. The cause of incorrectly
1 2 reporting this event was due to inadequate procedures. The procedures
1 3 are being revised. The revision will be completed prior to use for
1 4 vessel insulation removal. This is a final report.

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	G (28)	0 0 0 (29)	NA	A (31)	Work Package Review

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
1 9	Z (42)	NA

IE22

8308010301 830718
PDR ADOCK 05000416
S PDR

PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
2 0	N (44)	NA	

NAME OF PREPARER M. V. Rohrer PHONE:

GPO 917-926

SUPPLEMENTARY INFORMATION TO
LER 83-052/03 X-2

Mississippi Power & Light Company
Grand Gulf Nuclear Station - Unit 1
Docket No. 50-416

Technical Specification Involved: 3.4.6.1
Reported Under Technical Specification: 6.9.1.13.b

Event Narrative:

This is an update to previous reports submitted on May 11, 1983, and June 2, 1983. The event for which the reports were submitted is described in the following paragraphs.

On April 11, 1983, it was reported to the Shift Superintendent that the head/vessel flange thermocouples for monitoring the flange temperatures for compliance with Technical Specification 3.4.6.1 Surveillance Requirements (4.4.6.1.4) were not physically mounted on the vessel head/vessel flange surfaces. The event was reported by the I&C superintendent following a brief review of related maintenance work orders.

An investigation has revealed that the event was incorrectly reported. The vessel head/vessel flange thermocouples had been installed per work instructions and were actually monitoring flange temperatures. The flange thermocouples were reported as not having been installed due to inadequate installation procedures for the Reactor Vessel Head and the Vessel Head Insulation. These two separate documents do not currently address the installation/removal and the sequencing of the installation and removal of these thermocouples. The two craft organizations working on the installation misunderstood the sequencing of the installation and removal of these thermocouples.

Even if the thermocouples had not been installed, other means were available to verify the flange metal temperatures were greater than the 70°F Technical Specification limit. The reactor coolant temperatures and "ambient" air temperatures (assuming that the flange thermocouples were monitoring ambient air temperatures as originally reported) were being monitored during this time period and since the lowest "ambient" air temperature was 78°F and the lowest reactor coolant temperature was 85°F, based on the laws of thermodynamics and heat transfer, the flange metal temperatures were never less than 70°F. Table 1 (page 3) provides a listing of the temperatures recorded using the thermocouples and the reactor coolant temperatures during this time period.

In summary, the event should not have been reported. Our investigation revealed that related procedures were deficient and that revision is necessary. The revision of these procedures should prevent recurrence of this event.

In the previous report the estimated completion date for revision of related procedures was given as July 1, 1983. Revision was not made by the given date but procedure revision will be completed prior to use for vessel insulation removal. This is a final report.

VESSEL FLANGE/HEAD FLANGE

TABLE 1

DATE	Rx COOLAND TEMP (°F)	THERMOCOUPLES SET #1 (°F)	SET #2 (°F)	SET #3 (°F)
3/11	105	89/86	82/82	85/95
3/12	90	85/95	89/95	89/98
3/13	90	88/96	84/98	89/98
3/14	90	89/99	88/99	87/98
3/15	95	89/98	78/95	85/95
3/16	95	86/95	85/95	85/95
3/17	95	85/95	88/95	88/95
3/18	90	85/95	89/95	89/94
3/19	92	90/95	90/95	90/97
3/20	92	90/96	90/95	90/95
3/21	92	90/95	89/96	90/95
3/22	90	89/95	90/95	*/*
3/23	90	87/95	90/95	88/95
3/24	95	91/100	90/100	90/100
3/25	100	91/100	90/100	90/100
3/26	85	90/102	92/100	92/95
3/27	85	90/100	89/96	90/95
3/28	90	85/94	88/92	85/94
3/29	85	85/92	87/95	85/94
3/30	85	87/94	85/92	85/95
4/01	90	85/95	85/94	85/95
4/02	100	85/105	90/100	90/100
4/03	95	86/94	85/95	85/94
4/04	100	85/94	85/95	85/93
4/05	100	85/94	*/92	84/95
4/06	100	83/94	80/95	97/95
4/07	100	85/97	90/92	85/93
4/08	90	84/93	83/92	82/93
4/09	90	84/94	83/95	89/93
4/10	90	83/91	82/90	81/90

* No Data

MISSISSIPPI POWER & LIGHT COMPANY
Helping Build Mississippi
P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

July 18, 1983

JAMES P. McGAUGHY, JR.
VICE PRESIDENT

Office of Inspection & Enforcement
U. S. Nuclear Regulatory Commission
Region II
101 Marietta St., N.W., Suite 2900
Atlanta, Georgia 30303

Attention: Mr. J. P. O'Reilly, Regional Administrator

Dear Mr. O'Reilly:

SUBJECT: Grand Gulf Nuclear Station
Unit 1
Docket No. 50-416
License No. NPF-13
File 0260/L-835.0
Update Report - Reactor Vessel
Flange and Head Flange
Temperature Surveillance
Requirements Not Met
LER 83-052/03 X-2
AECM-83/0401

This letter submits an update to previous reports submitted on May 11, 1983, and June 2, 1983. The event for which the reports were submitted occurred on April 11, 1983, when it was discovered that the head flange and vessel flange thermocouples had not been physically reinstalled during the insulation installation. Therefore, it was thought the surveillance requirements of Technical Specification 4.4.6.1.4 were not met. The event was reported pursuant to Technical Specification 6.9.1.13.b.

In Revision 1 to LER 83-052 we reported that after evaluation of the event a determination was made that the event was incorrectly reported in Revision 0, The thermocouples had actually been installed but due to a misunderstanding were reported as not being installed. We also stated that procedures were being revised to prevent recurrence and revision was expected by July 1, 1983.

Revision was not made by July 1, 1983. Procedure revision will be completed prior to use for vessel insulation removal. Attached is LER 83-052/03 X-2, which is a final report.

Yours truly,

L. F. Dale
for J. P. McGaughy

JPM:sap
Attachment

cc: (See Next Page)

IE 22
1/1

MISSISSIPPI POWER & LIGHT COMPANY

cc: Mr. J. B. Richard (w/a)
Mr. R. B. McGehee (w/o)
Mr. T. B. Conner (w/o)
Mr. G. B. Taylor (w/o)

Mr. Richard C. DeYoung, Director (w/a)
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