PUBLIC SERVICE COMPANY OF COLORADO

P. O. BOX 840 · DENVER, COLORADO 80201

OSCAR R. LEE
VICE PRESIDENT

July 20, 1983
Fort St. Vrain
Unit No. 1
P-83242

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

DOCKET NO. 50-267

SUBJECT: Fort St. Vrain Updated FSAR,
Revision 1

Dear Sir.

Enclosed is one (1) signed original and twelve (12) additional copies of Revision 1 to the Updated Final Safety Analysis Report (Updated FSAR) for Fort St. Vrain which has been prepared and is being submitted in accordance with 10CFR50, Section 50.71(e).

This revision includes the effects of changes made in the facility or procedures as described in the Updated FSAR; safety evaluations performed by PSC, either in support of license amendments or in support of conclusions that changes did not involve an unreviewed safety question; and analyses of new safety issues performed by or on behalf of PSC at Commission request.

Brief descriptions of the principal changes or additions to affected sections and appendices of the Updated FSAR, Revision 1, are included in Attachment A. Other changes made in the FSAR, not requiring prior Commission approval pursuant to 10CFR50.59(a), and not yet reported to the Commission, are listed in Attachment B to this letter. These changes will be more fully described along with the results of their safety evaluations, as appropriate, in PSCo's regular report of 10CFR50.59 changes which will be submitted in early 1984.

Very truly yours,

ORLee

O. R. Lee, Vice President
Electric Production

ORL/DCG:pa
Attachments

A - Description of Principal Changes Incorporated into Revision 1 of the Updated FSAR
B - List of 10CFR50.59 changes not previously reported to the Commission

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APPENDIX A

Added a brief discussion of the results of the nondestructive examination of the first fuel test element (FTE-1).

APPENDIX B

Added new information pertaining to the certification and use of ASTM A-36 steel. Added information regarding independent verification of the condition/position of critical equipment prior to return to service following clearance conditions as contained in previous PSC responses to NUREG-0737. Incorporated those minor changes to or clarifications of the quality assurance program previously reported to the Commission in letter P-83203 (dated June 9, 1983).

APPENDIX C

Updated design criteria to reflect the steam generator penetration interspace pressure control modification.

APPENDIX D

Added a brief evaluation of control room habitability and reactor building accessibility for post accident conditions as contained in previous PSC responses to NUREG-0737.

NOTE: Where appropriate, sections were revised to use consistant terminology or make editorial clarification of descriptive material to achieve consistency with other sections in the FSAR. Minor typographical and editorial errors were also corrected.

SECTION 7

Revisions to this section added information concerning radiation monitoring of the steam generator penetration interspace, the addition of a sixty meter meteorological tower and associated instrumentation and a discussion of the first plateout probe removal.

SECTION 9

Incorporated information concerning control of heavy loads previously submitted to the Commission.

SECTION 10

Incorporated ISI program information pertaining to the secondary coolant system valves.

SECTION 11

Updated the discussion concerning operation of the radioactive liquid waste system per 10CFR20, including use of the reactor building sump as part of this system (as requested by NRC Region IV in IE Inspection Report 83-06 dated May 23, 1983). Included information concerning post accident sampling contained in previous PSC responses to NUREG-0737.

SECTION 12

Updated the information pertaining to the radiological emergency response plan and early warning system.

SECTION 13

Added new material discussing the ISI program in general and updated the summary of the related surveillance requirements.

SECTION 14

Added updated information concerning masonry block walls and addressed the possibility of primary coolant leakage into the secondary coolant system via the steam generator penetration interspaces. Added a brief discussion of the validation of the Reactor Emergency Cooling Analysis (RECA-3) code.

Attachment A
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Description of Principal Changes Incorporated in Revision 1 to the Updated FSAR

SECTION 1

Brief descriptions have been added to the design modification summary portion of this section to address the steam generator interspace pressure control modification and meteorological equipment modifications.

SECTION 3

Information was added to incorporate the comparison region method of reactor operation as approved by the Commission in Amendment 28 to the Facility Operating License. A discussion of the first plateout probe removal and examination was inserted. New material was added concerning the In-Service Inspection and Testing (ISI) program pertaining to the reactor internals. The results of the visual examination of the two fuel elements with graphite block web cracks were incorporated. A discussion of the reserve shutdown system previously located in Section 4 was moved to Section 3 to improve continuity.

SECTION 4

A discussion of the steam generator penetration interspace pressure control modification was added. New information was added concerning the ISI program as related to the helium circulators and shutoff valves. A modification to the helium circulator steam/water drains was incorporated and information pertaining to the reserve shutdown system was moved to Section 3.

SECTION 5

New sections were added concerning the ISI program pertaining to PCRV closures and penetrations.

SECTION 6

Several sections were updated to reflect the current ISI program and its applicability to the steam/water dump system, PCRV penetration secondary closures and flow restriction devices, and the PCRV safety valves.

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter	)	
	)	
Public Service Company of Colorado	)	Docket No. 50-267
Fort St. Vrain Unit No. 1	)	

AFFIDAVIT

O. R. Lee, being duly sworn, hereby deposes and says that he is Vice President of Public Service Company of Colorado; that he is duly authorized to sign and file with the Nuclear Regulatory Commission Revision 1 of the Updated FSAR; that he is familiar with the content thereof; and that the matters set forth therein are true and correct to the best of his knowledge, information and belief.

ORLee
O. R. Lee, Vice President

STATE OF Colorado)
COUNTY OF Denver)

Subscribed and sworn to before me, a Notary Public in and for _____ _____ on this 19th day of July, 1983.

Ira LeBlanc
Notary Public
4026 E. 113th Place
Thornton, Co 80233

My commission expires August 19, 1983.

Attachment B
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FSAR CHANGES PER 10CFR50.59
(Not Previously Reported to the Commission)

As required by 10CFR50, Section 50.71(e-2(ii)), following is a list of the changes made under the provisions of 10CFR50, Section 50.59 which have not previously been submitted to the Commission. These changes will be more fully described, along with the results of their safety evaluations, as appropriate, in PSC's regular report of 10CFR50.59 changes which will be submitted in early 1984.

FSAR Section	Description of Change
7.2	Control rod position information was removed from a multipoint recorder and is now processed by the data logger computer.
7.4	Hand switches were installed on the auxiliary control board to permit manual control of the main steam bypass valves in the event of abandonment of the control room.
11.1	A procedure was revised to permit utilization of the Reactor Building Sump for dilution of radioactive liquid wastes in the event such dilution is necessary to meet the requirements of 10CFR20.