

Report of Changes, Tests, and Experiments
Performed at Pilgrim Nuclear Power Station

The following changes and tests were performed at the Pilgrim Nuclear Power Station between July 1, 1982 and January 21, 1983, under the authorization of 10CFR50.59(a). No experiments were performed at the Pilgrim Nuclear Power Station during this period. For each change or test reported below, the statement that it does not involve an unreviewed safety question indicates that the change or test was reviewed prior to implementation and that it was determined that:

1. The change or test would not increase the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report;
2. The change or test would not create a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report; and
3. The change or test would not reduce the margin of safety as defined in the basis for any technical specification.

PLANT CHANGES

PDCR 78-42

Area Rad Monitors

This PDCR provided area radiation monitors with remote readout capabilities in the radwaste control room for the spent resin, flatbed filter liners, and radwaste demineralizer areas.

Ref. FSAR Section 7.13.5

Safety Evaluation No. 595

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 78-45

TIP Room Area Rad Monitors

This PDCR added a local area radiation monitoring system to the Traversing Incore Probe (TIP) room and the Condensate Regeneration room.

Ref. FSAR Section 7.13.5

Safety Evaluation No. 79-SE-01

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 79-M-08

Instrument and Control Lab Mods

Due to an increase in staff and equipment, additional space was needed. This change relocated storage of some I&C equipment, redesigned the present lab, and created a smaller lab for additional space. (Note: This PDCR has been superseded and the lab relocated in preparation for the new Administration Building presently under construction).

Ref. FSAR Section 12.1

Safety Evaluation No. 79-SE-16

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 79-27F

AOG Information Retrieval

This PDCR provided on demand monitoring of various Augmented Off Gas (AOG) system activities in the Main Control Room. Through the installation of thermocouples, steam flow transmitters and various electronics, the control room operators are now able to monitor steam conditions prior to entry to the AOG system and various in-system conditions previously monitored from the recombiner area.

Ref. FSAR Section 9.4.4.1.6

Safety Evaluation No. 80-PSG-2

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 79-59

Torus Level Monitoring

To fulfill the requirements of NUREG-0578, two channels of redundant level instrumentation capable of monitoring the torus level were added.

Ref. FSAR Section T-5.2.3

Safety Evaluation No. 1008

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 80-58

Installation of EOF Communication

This PDCR is one of several which incorporated modifications due to TMI. The governing documents for the work covered by this PDCR are NUREG 0660 "NRC Action Plan Developed as a Result of the TMI-2 Accident" and NUREG 0696 "Functional Criteria for Emergency Response Facilities."

Modifications covered by this PDCR include, circuitry to transmit meteorological data to the EOF, communications (including additional land lines to Civil Defense Centers), Emergency power, security devices (including alarms, forces and outside lighting) and fire detectors.

Ref. FSAR Section Appendix P

Safety Evaluation No. 1047

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 81-04C

Torus Modifications (Monorail)

This change removed the monorail and its supports from the Torus compartment and placed in storage in the Torus room. This modification conforms to NRC Acceptance Criteria, Appendix A to NUREG-0661.

Ref. FSAR Section Figures 12.1-13, 15

Safety Evaluation No. 1127

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 81-16

Control Rod Drive Temp. Recorder

The original temperature recorder (Leeds & Northrup) was worn out and obsolete. It was replaced with a Kaye Instrument, all solid state data logger.

Ref. FSAR Section Table 7.7-1

Safety Evaluation No. 1117

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

PDCR 81-46

HPCI Turbine Ramp Generator

This change replaces the "anti-wind up" circuits in the HPCI flow controller loop with a "ramp generator" module in the turbine control system.

All equipment and material are safety-related. The equipment purchased for this change has been accepted as qualified to IEEE-323-1974 and IEEE-344-1975.

Ref. FSAR Section Figure 7.4-5

Safety Evaluation No. 1318

This change does not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

The following PDCR's (79-24 Series), modified hangers and supports in various locations, in response to IE Bulletin 79-14.

Reference FSAR Sections 2.5.3 and 12.2.3.5.2

These changes do not involve an unreviewed safety question as defined in 10CFR-50.59(a) or a change to a technical specification.

<u>PDCR No.</u>	<u>Safety Evaluation No.</u>	<u>Hanger/Mark No.</u>
79-24A	672	H-30-1-41-SG H-30-1-41-SR H-30-1-44-SG
79-24-F.3	739	H-26-1-135
79-24-K.12	1222	H-30-1-92 H-30-1-6SS H-30-1-80 H-30-1-109 H-30-1-111SR H-30-1-138
79-24-N.11	1193	H-30-1-357 H-30-1-66SS H-30-1-328 H-30-1-286 H-30-1-308 H-30-1-369
79-24-T.6	1153	H-30-1-296 H-30-1-317 H-30-1-58SA H-30-1-340 H-30-1-SA3 H-30-1-64SA H-30-1-355 H-30-1-367
79-24-XE.12	931	H-30-1-114SR H-30-1-22SG
79-24-XE.13	961	H-30-1-10SS
79-24-XE.15	988	H-26-1-194
79-24-XE.16	993	H-30-1-110
79-24-XE.17	931	H-30-1-58 H-30-1-113 H-30-1-142 H-30-1-72 H-30-55SG

<u>PDCR No.</u>	<u>Safety Evaluation No.</u>	<u>Hanger/Mark No.</u>
79-24-XF.1	798	H-30-1-238
79-24-XF.2	821	H-30-1-171 H-30-1-175 H-30-1-174
79-24-XF.3	830	H-30-1-SA-6 H-30-1-104SA H-30-1-96SA
79-24-XF.4	844	H-14-1-34
79-24-XF.5	857	H-30-1-1219
79-24-XF.6	875	H-30-1-239
79-24-XF.7	878	H-14-1-33
79-24-XF.8	893	H-14-1-1221
79-24-XF.9	933	H-30-1-1218
79-24-XF.10	980	H-30-1-173
79-24-XG.1	813	H-10-1-129
79-24-XH.1	866	H-19-1-SA
79-24-XJ.1	837	H-6-1-59 H-1-1-1210
79-24-XJ.2	861	H-6-1-66 FG-1-15R FG-1-14R H-6-1-67
79-24-XJ.3	868	FG-1-13R FG-1-12R
79-24-XJ.4	882	H-6-1-64
79-24-XJ.5	939	H-6-1-58
79-24-XJ.6	955	F-6-1-10R F-6-1-11R
79-24-Y.13	872	H-30-1-60SR H-30-1-73SH H-30-1-376

The following PDCR's (81-53 Series) modified masonry walls in various locations, in response to IF Bulletin 80-11.

Reference FSAR Sections 12.2

These changes do not involve an unreviewed safety question as defined in 10CFR-50.59(a) or a change to a technical specification.

<u>PDCR No.</u>	<u>Safety Evaluation No.</u>	<u>Wall No.</u>
81-53-AB.1	1285	185.2S
81-53-AB.2	1307	185.11 E&W
81-53-AC.3	1323	188.9 N&S 188.11 E&W
81-53-RB.13	1327	65.2S
81-53-RB.4	1242	64.13 N&S
81-53-RC.3	1265	68.2 N 68.1 E&W 68.4 N 68.3 N
81-53-RC.4	1275	66.11 N&S 66.12 N&S 66.18 E&W 68.0 N&S
81-53-TB.3	1313	209.8 N&S
81-53-TD.3	1316	210.2 E&W
81-53-WA-3	1284	191.55 N&S 191.56 E&W
81-53-WA.6	1306	191.35 S
81-53-WA.5	1332	191.25 E&W
81-53-WA.7	1326	191.46 W
81-53-WA.8	1329	191.49 E&W
81-53-WC.1	1283	194.20 N&S
81-53-WC.2	1303	194.23 S
81-53-WC.3	1310	194.24 N&S

PDCR No.

Safety Evaluation No.

Wall No.

81-53-WC.4

1325

194.22 E&W

81-53-WC.5

1333

194.21 E&W

PLANT TESTS

TP-82-25

To monitor performance of the SSW system and RBCCW Heat Exchanger to assure that each operable loop of the SSW system can achieve a cooling capability equivalent to 5000 GPM flow at 65°F inlet SSW temperature through the respective RBCCW Heat Exchanger with the associated TRCCW Heat Exchanger throttled. This test will also be used to develop procedural guidance for backwashing and cleaning RBCCW Heat Exchangers.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

TP-82-32

The purpose of this procedure is to safely checkout and test the installation and operation of the new low level Radwaste drum-type compactor, prior to turnover to Waste Management.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

TP-82-34

The purpose of this procedure is the determination of the existence of air pockets in the Reactor Water Level instruments and associated piping.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

TP-82-42

The new B condensate pump is one of the unit 2 pumps modified by the manufacturer for use on unit one. The motor on the pump is the original motor for the old B condensate pump.

Since this is not an in kind replacement and the calculated pump curve is shaped differently than the curves generated for A and C condensate pumps, special care must be exercised until B condensate pumps characteristics, and its ability to run in parallel with the existing pumps, are demonstrated.

The purpose of this procedure is to provide guidance for the initial start-up and testing of B condensate pump.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

TP-82-45

To provide a detailed instruction for operating personnel to perform a flow and integrity test of the off-site shipment line integrity using condensate transfer system.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

TP-82-51

The purpose of this procedure is to collect data concerning the reactor water level instrumentation during startup of the reactor.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

TP-82-53

To provide a method for proving administratively, SSW system operability in a manner more consistent with the FSAR Heat Removal Criteria. This criteria of Section VI requires that a SSW Loop deliver 5000 gpm flow at an inlet temperature of 65°F (equivalent).

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

TP-82-75

To provide instructions for performing an RCIC system operability test with reactor pressure < 150 psig.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

TP-82-80

To provide a detailed instructions for station personnel to perform a test to determine possible effects on reactor water conductivity and main steam line radiation when CRD system suction is transferred from condensate feedwater header to the condensate storage tank.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.

TP-82-5

A test which will assist in diagnosing the movement problems being encountered with control rods as well as attempting to move the rods out to position 48.

This test did not involve an unreviewed safety question as defined in 10CFR50.59(a) or a change to a technical specification.