

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 ALBR13200-000000-000341111145
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

CONT

01 REPORT SOURCE L6056000296706208380719839
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 With unit 3 operating normally, while performing SI 4.7.D.1.b-1, Primary Con-
03 tainment Isolation Valves Operability Test, valve FCV-69-2 would not fully close
04 (T.S. 3.7.D.1). Valve FCV-69-1 was closed per T.S. 3.7.D.2. FCV-69-2 was
05 inoperable for 4 days and 13.5 hours. There was no effect on public health and
06 safety. T.S. 3.7.D.2 allows continuous operation with redundant valve FCV-69-1
07 closed.08
09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
S D 11 C 12 A 13 V A L V O P 14 B 15 Z 16
17 LER/NO REPORT NUMBER 8 3 21 22 23 24 25 26 27 28 29 30 31 32 REVISION NO. 0
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS 22 ATTACHMENT SUBMITTED FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER 26
A 18 Z 19 Z 20 Z 21 0 0 0 0 0 Y 23 N 24 N 25 P 2 9 6 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Valve FCV-69-2 would not close due to the valve operator motor breaker tripping.
11 Breaker tripped due to armature winding failure in Limitorque motor (Porter S/N
12 Ez28903, DK56H frame. 1.8 HP, 250VDC, 5 min. duty, 7.5 Amps, 1900 RPM). Motor
13 was replaced. This is considered a random failure and no recurrence control is
14 required.15 FACILITY STATUS % POWER OTHER STATUS 30 METHOD OF DISCOVERY DISCOVERY DESCRIPTION 32
E 28 0 9 9 29 NA B 31 Surveillance tests
16 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36
Z 33 Z 34 NA NA
17 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39 NA
0 0 0 37 Z 38
18 PERSONNEL INJURIES NUMBER DESCRIPTION 41 NA
0 0 0 40
19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION 43 NA
Z 42
20 PUBLICITY ISSUED DESCRIPTION 45 NA
N 448307260552 830719
PDR ADOCK 05000296
S PDR

NAME OF PREPARER Walter Christopher

PHONE (205) 729-0889

NRC USE ONLY

LER SUPPLEMENTAL INFORMATION

BFRO-50- 296 / 83035 Technical Specification Involved 3.7.D.1 and 3.7.D.2

Reported Under Technical Specification 6.7.2.b.(2) * Date Due NRC 7/20/83

Event Narrative:

Unit 1 was in a refueling outage; unit 2 was operating normally at 98 percent power; unit 3 was operating normally at 99 percent power. Unit 3 was the only unit affected by this event. On June 20, 1983, while performing Surveillance Instruction 4.7.D.1.b-1 (Primary Containment Isolation Valves Operability Test), valve FCV-69-2 would not fully close. Valve FCV-69-1 was closed (mode corresponding to isolated condition) per Technical Specification 3.7.D.2. Investigation revealed valve No. FCV-69-2 had an armature winding failure in Limitorque motor (Porter S/N EZ28903 DK56H frame, 1.8 HP, 250VDC, 5 min. duty, 7.5 Amp, 1900 RPM). The Limitorque motor was replaced. There was no effect on public health and safety. Redundant valve FCV-69-1 was maintained in the closed position until FCV-69-2 was returned to service as specified by Technical Specification 3.7.D.2. This is considered a random failure and no recurrence control is required.

* Previous Similar Events:

None

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

TENNESSEE VALLEY AUTHORITY

USNRC REGION 1
CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

83 JUL 21 A 9:49

July 19, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET
NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE
REPORT BFRO-50-296/83035

The enclosed report provides details concerning a primary containment
isolation valve which would not fully close during surveillance testing.
This report is submitted in accordance with Browns Ferry unit 3 Technical
Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY



H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

OFFICIAL COPY

TE 22 //