

CONTROL BLOCK

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 N C M G S 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CONT

01 REPORT SOURCE L 0 5 0 0 0 3 7 0 0 6 0 9 8 3 0 7 1 8 8 3 3
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 While in Mode 1, following maintenance to its actuator, valve 2CF-35 (S/G 2A
03 feedwater containment isolation) exceeded its stroke time of ≤ 5 seconds. Valve
04 2NC53 (nitrogen to pressurized relief tank #2, containment isolation outside)
05 was also declared inoperable when control room status lights and the computer
06 indicated it failed to close. These violate T.S.3.6.3 which is reportable per
07 T.S.6.9.1.11(d). The valves were closed and deenergized, thus performing their
08 safety function. Redundant valves were available. Health and safety of the
09 public were unaffected.

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP SUBCODE VALVE SUBCODE
S D 11 E 12 X 13 V A L V O P 14 C 15 Z 16
17 LEAKAGE REPORT NUMBER 8 3 0 2 7 0 3 L 0
18 ACTION TAKEN 19 FUTURE ACTION 20 EFFECT ON PLANT 21 SHUTDOWN METHOD 22 HOURS 23 ATTACHMENT SUBMITTED 24 NPRO-4 FORM SUB. 25 PRIME COMP. SUPPLIER 26 COMPONENT MANUFACTURER
A 18 Z 19 Z 20 Z 21 0 0 0 0 N 23 N 24 L 25 B 3 5 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Valve failures are attributed to component failure due to solenoid failure of
11 the operator of valve 2CF35 and component malfunction for the failure of 2NC53
12 to close (or indicate close). The solenoid valves of 2CF35 were replaced. Valve
13 2NC53 was leak rate tested and cycled several times with no apparent problems.

14
15 FACILITY STATUS 16 % POWER 17 OTHER STATUS 18 METHOD OF DISCOVERY 19 DISCOVERY DESCRIPTION
B 28 0 3 0 29 N/A B 31 Routine Surveillance 32

16 ACTIVITY CONTENT 17 AMOUNT OF ACTIVITY 18 LOCATION OF RELEASE
Z 33 Z 34 N/A 35 N/A 36

17 PERSONNEL EXPOSURES 18 TYPE 19 DESCRIPTION
0 0 0 37 Z 38 N/A 39

18 PERSONNEL INJURIES 19 DESCRIPTION
0 0 0 40 N/A 41

19 LOSS OF OR DAMAGE TO FACILITY 20 TYPE 21 DESCRIPTION
Z 42 N/A 43

20 PUBLICITY 21 ISSUED DESCRIPTION
N 44 N/A 45

NAME OF PREPARER Phillip B. Nardoci

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S PDR

NRC USE ONLY

DUKE POWER COMPANY

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HAL B. TUCKER
VICE PRESIDENT
NUCLEAR PRODUCTION

July 18, 1983

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U.S. NRC REGION II
ATLANTA, GEORGIA

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street NW, Suite 2900
Atlanta, Georgia 30303

Subject: McGuire Nuclear Station Unit 2
Docket No. 50-370
LER/RO-370/83-27

Dear Mr. O'Reilly:

Please find attached Reportable Occurrence Report RO-370/83-27. This report concerns T.S. 3.6.3, "The containment isolation valves specified in Table 3.6-2 shall be operable with isolation times as shown in Table 3.6-2". This incident was considered to be of no significance with respect to the health and safety of the public.

Very truly yours,

H.B. Tucker / *HT*
Hal B. Tucker

PBN:jfw
Attachment (1)

cc: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. W. T. Orders
NRC Resident Inspector
McGuire Nuclear Station

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Institute of Nuclear Power Operations
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