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J. D. Woodard  
Vice President-Nuclear  
Farley Plant

May 13, 1991



Docket No. 50-364

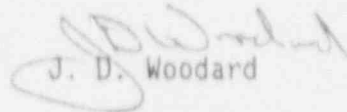
U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D. C. 20555

Joseph M. Farley Nuclear Plant  
Unit 2  
Monthly Operating Data Report

Attached is the April 1991 Monthly Operating Report for Joseph M. Farley Nuclear Plant Unit 2, as required by Section 6.9.1.10 of the Technical Specifications.

If you have any questions, please advise.

Respectfully submitted,

  
J. D. Woodard

AEJ:edb3014

Attachments

cc: Mr. S. D. Ebnetter  
Mr. S. T. Hoffman  
Mr. G. F. Maxwell

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JOSEPH M. FARLEY NUCLEAR PLANT  
UNIT 2  
NARRATIVE SUMMARY OF OPERATIONS  
April 1991

There were three reactor trips during the month of April.

At 1055 on 4-1-91, while operating at approximately 100 percent power, a reactor trip occurred when rod H-10 dropped into the core. The reactor trip occurred due to a high negative flux rate as detected by the power range nuclear detectors. The operator was performing FNP-2-STP-5.0 (Full Length Control Rod Operability Test). When control rod group C was tested, rod H-10 dropped into the core. This event was caused by defective circuit card(s) in the rod control system. The suspect cards were replaced and the unit returned to power operation at 1208 on 4-9-91.

At 2040 on 4-9-91, while operating at approximately 34 percent power, the reactor was manually tripped following the loss of the operating (2A) steam generator feedwater pump (SGFP). The loss of the SGFP was caused by a failed electro-hydraulic (EH) fluid supply line coupling and subsequent loss of EH fluid. Examination of the failed tubing revealed that the failure resulted from cyclic fatigue. The EH tubing was repaired and all accessible welds on the EH systems on Units 1 and 2 SGFPs were inspected. Vibration readings were taken on the EH tubing in the vicinity of the failure. The failed coupling was repaired and an investigation is continuing to determine if additional corrective action is necessary. The unit returned to power operation at 1648 on 4-10-91.

At 1616 on 4-20-91, while operating at approximately 100 percent power, the Unit two reactor tripped due to turbine generator trip caused by a loss of condenser vacuum. The condenser lost vacuum because an instrument air valve which supplies air to the steam admission valve on "A" steam jet air ejector was inadvertently closed. The valve was closed as workers were rolling up an air hose after completing a work assignment in the area. The unit returned to power operation at 0426 on 4-21-91.

There were no other unit shutdowns or major power reductions during the month of April.

The following major safety-related maintenance was performed during the month of April:

1. Defective circuit card(s) were replaced in the rod control system.

# OPERATING DATA REPORT

DOCKET NO. 50-364

DATE May 7, 1991

COMPLETED BY D. N. Morey

TELEPHONE (205)899-5156

## OPERATING STATUS

1. Unit Name: Joseph M. Farley - Unit 2
2. Reporting Period: April 1991
3. Licensed Thermal Power (MWt): 2,652
4. Nameplate Rating (Gross MWe): 860
5. Design Electrical Rating (Net MWe): 829
6. Maximum Dependable Capacity (Gross MWe): 864.2
7. Maximum Dependable Capacity (Net MWe): 824.0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: N/A
9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

### Notes

- 1) Cumulative data since 7-30-81, date of commercial operation.

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	719	28.9	85,488
12. Number Of Hours Reactor Was Critical	525.2	2611.9	73,003.1
13. Reactor Reserve Shutdown Hours	0.0	0.0	138.0
14. Hours Generator On-Line	494.5	2524.0	72,075.2
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,237,745	6,412,630	183,616,579
17. Gross Electrical Energy Generated (MWH)	398,978	2,093,216	60,299,172
18. Net Electrical Energy Generated (MWH)	372,504	1,980,948	57,175,070
19. Unit Service Factor	68.8	87.7	84.3
20. Unit Availability Factor	68.8	87.7	84.3
21. Unit Capacity Factor (Using MDC Net)	62.9	83.5	81.7
22. Unit Capacity Factor (Using DER Net)	62.5	83.0	80.7
23. Unit Forced Outage Rate	31.3	8.2	4.4
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):	N/A		

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	05/06/81	05/08/81
INITIAL ELECTRICITY	05/24/81	05/25/81
COMMERCIAL OPERATION	08/01/81	07/30/81

DOCKET NO. 50-364UNIT 2DATE May 7, 1991COMPLETED BY D. N. MoreyTELEPHONE (205)899-5156MONTH April

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>365</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>37.5</u>
10	<u>23</u>
11	<u>320</u>
12	<u>802</u>
13	<u>820</u>
14	<u>824</u>
15	<u>823</u>
16	<u>823</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>824</u>
18	<u>826</u>
19	<u>826</u>
20	<u>549</u>
21	<u>449</u>
22	<u>827</u>
23	<u>822</u>
24	<u>831</u>
25	<u>830</u>
26	<u>826</u>
27	<u>820</u>
28	<u>816</u>
29	<u>816</u>
30	<u>823</u>

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-364  
 UNIT NAME J. M. FURLEY - UNIT 2  
 DATE May 7, 1991  
 COMPLETED BY D. N. MOREY  
 TELEPHONE (205)899-5156

REPORT OF APRIL

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR <sup>3</sup>	LICENSEE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
004	910401	F	192.2	A	3	91-001-00			At 1055 on 4-1-91, while operating at approximately 100 percent power, a reactor trip occurred when rod H-10 dropped into the core. The reactor trip occurred due to a high negative flux rate as detected by the power range nuclear detectors. The operator was performing FNP-2-STF-3.0 (Full Length Control Rod Operability Test). When control rod group C was tested, rod H-10 dropped into the core.  This event was caused by defective circuit card(s) in the rod control system. The suspect cards were replaced and the unit returned to power operation at 1208 on 4-09-91.
005	910409	F	20.1	A	2	91-002-00			At 2040 on 4-9-91, while operating at approximately 34 percent power, the Unit two reactor was manually tripped following the loss of the operating (2A) steam generator feedwater pump (SGFP). The loss of the SGFP was caused by a failed electro-hydraulic (EH) fluid supply line coupling and subsequent loss of EH fluid. The failure was in the heat affected zone of a tube-to-fitting socket weld on the EH supply to the high pressure governor valve to the 2A SGFP. Examination of the failed tubing revealed that the failure resulted from cyclic fatigue.

<sup>1</sup> F: Forced  
 S: Scheduled

<sup>2</sup> Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup> Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup> Exhibit G-Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report(LER) File (NUREG-  
 0161)

<sup>5</sup> Exhibit I -Same Source

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-364  
 UNIT NAME J. M. FARLEY - UNIT 2  
 DATE May 7, 1991  
 COMPLETED BY D. N. MOREY  
 TELEPHONE (205)899-5156

REPORT MONTH APRIL

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR <sup>3</sup>	LICENSEE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
005 (continue)									The EH tubing was repaired and all accessible welds on the EH systems on Units 1 and 2 SGFPs were inspected. Vibration readings were taken on the EH tubing connections in the vicinity of the Unit 2 SGFP turbines. Vibration was found on the EH tubing in the vicinity of the failure. The highest vibration occurred during low power plant operation. An investigation is continuing to determine if additional corrective action is necessary.
									The unit returned to power operation at 1648 on 4-10-91.
006	910420	F	12.2	H	3	91-004-00			At 1616 on 4-20-91, while operating at approximately 100 percent power, the Unit 2 reactor tripped due to turbine generator trip caused by a loss of condenser vacuum. The condenser lost vacuum because an instrument air valve which supplies air to the steam admission valve on the JAE was inadvertently closed. The valve was closed as workers were rolling up an air hose after completing their work assignment in the area.
									The unit returned to power operation at 0426 on 4-11-91.

<sup>1</sup> F: Forced  
 S: Scheduled

<sup>2</sup> Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup> Method:  
 1-Manual  
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 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup> Exhibit G-Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report(LEF) File (NUREG-  
 0161)

<sup>5</sup> Exhibit I -Same Source

(9/77)