

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01SPS200-00000-0034111145

REPORT SOURCE L05000281706228380711839

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

012 With the unit at 100% power, IRPI for control rod K-2 was fluctuating greater than
013 12 steps. This is contrary to T.S.-3.12.E.1 and is reportable per T.S.-6.6.2.b.(2).
014 The malfunctioning indication did not affect the control of the control rod and the
015 rod was capable of being dropped. Therefore, the health and safety of the public
016 were not affected.
017
018

SYSTEM CODE RB11 CAUSE CODE E12 CAUSE SUBCODE E13 COMPONENT CODE INSTRU14 COMP. SUBCODE E15 VALVE SUBCODE Z16
 LER/RO REPORT NUMBER 83 EVENT YEAR 83 SEQUENTIAL REPORT NO. 031 OCCURRENCE CODE 03 REPORT TYPE L REVISION NO. 0
 ACTION TAKEN X18 FUTURE ACTION X19 EFFECT ON PLANT Z20 SHUTDOWN METHOD Z21 HOURS 000000 ATTACHMENT SUBMITTED Y23 NRC-4 FORM SUB. Y24 PRIME COMP. SUPPLIER N25 COMPONENT MANUFACTURER W35126

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

110 The exact cause has not been determined. Evaluation to date indicates that the
111 problem may have been originating from the containment. The signal conditioner
112 module voltage was adjusted and the fluctuations have ceased, however further testing
113 will be conducted during the present refueling outage.
114

FACILITY STATUS E28 % POWER 10029 OTHER STATUS N/A30 METHOD OF DISCOVERY A31 DISCOVERY DESCRIPTION Operator Observation32
 ACTIVITY CONTENT Z33 RELEASED OF RELEASE Z34 AMOUNT OF ACTIVITY N/A35 LOCATION OF RELEASE N/A36
 PERSONNEL EXPOSURES NUMBER 00037 TYPE Z38 DESCRIPTION N/A39
 PERSONNEL INJURIES NUMBER 00040 DESCRIPTION N/A41
 LOSS OF OR DAMAGE TO FACILITY TYPE Z42 DESCRIPTION N/A43
 PUBLICITY N44 DESCRIPTION N/A45

8307250485 830711
 PDR ADOCK 05000281
 S PDR

NRC USE ONLY

NAME OF PREPARED J. L. Wilson

PHONE: (804) 357-3184

ATTACHMENT 1

SURRY POWER STATION, UNIT NO. 2

DOCKET NO: 50-281

REPORT NO: 83-031/03L-0

EVENT DATE: 06-22-83

TITLE OF THE EVENT: ERRATIC ROD POSITION INDICATION

1. Description of the Event

With the unit at full power, operators noted the IRPI for control rod K-2 indicating erratically and fluctuating greater than 12 steps, thereby rendering the indication unreliable. This event is contrary to Technical Specification 3.12.E.1 and is reportable per Technical Specification 6.6.2.b.(2).

2. Probable Consequences and Status of Redundant Equipment

The Rod Position indicators give continuous indication of the position of each control rod. The RPI fluctuation did not affect the control of Rod K-2 and it was capable of being dropped, therefore the health and safety of the public were not affected.

3. Cause

The cause of the event has not been determined. It is suspected that an erroneous signal to the IRPI was being generated inside the containment.

4. Immediate Corrective Action

That portion of the system outside the containment was checked and the signal conditioner module voltage was discovered to be high but stable. The voltage was adjusted to its proper setting and fluctuations of the indicator ceased.

5. Subsequent Corrective Action

The IRPI has been under observation and no further erratic behavior has been observed, however, the portion of the system inside the containment will undergo further testing during the present outage.

6. Action Taken to Prevent Recurrence

The results of the testing during the outage will determine if any action will be required.

7. Generic Implications

None.

Vepco

USNRC REGION II
ATLANTA, GEORGIA

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VIRGINIA ELECTRIC AND POWER COMPANY

Surry Power Station
P. O. Box 315
Surry, Virginia 23883

Serial No: 83-050

Docket No: 50-281

License No: DPR-37

Mr. James P. O'Reilly
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly

Pursuant to Surry Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 2.

Report Number

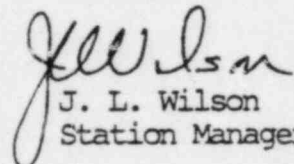
83-031/03L-0

Applicable Technical Specification

T. S. 6.6.2.b(2)

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,


J. L. Wilson
Station Manager

Enclosure

cc: Document Control Desk, USNRC
016 Phillips Bldg.
Washington, D. C. 20555

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