

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99

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LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

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REPORT SOURCE L 6 0 5 0 0 0 2 8 0 7 0 6 1 6 8 3 8 0 7 1 1 8 3 9

DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99

On June 16 and again on June 17, 1983, the containment partial air pressure was at a value greater than that stipulated in T.S. figure 3.8.1. This event is contrary to T.S.-3.8.A.4 and is reportable per T.S.-6.6.2.b.(2). The containment pressure remained less than .25 psi above the allowable value. Hence, the containment could have performed its designed function. Therefore, the health and safety of the public were not affected.

0 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

S A 11 X 12 Z 13 V E S S E L 14 D 15 Z 16

LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.

8 3 0 2 6 0 3 L 0

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NFRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

X 18 Z 19 Z 20 Z 21 0 0 0 0 0 Y 23 N 24 A 25 S 4 2 0 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 1 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99

Service water temperature, which is one of the parameters that determines the allowable containment pressure, was increasing faster than containment pressure was decreasing. The net result was that allowable pressure decreased below actual containment pressure.

1 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99

FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION

C 28 0 3 5 29 N/A 30 A 31 Observation 32

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE

Z 33 Z 34 N/A 35 N/A 36

PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION

0 0 0 37 Z 38 N/A 39

PERSONNEL INJURIES NUMBER DESCRIPTION

0 0 0 40 N/A 41

LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION

Z 42 N/A 43

1 9 2 0 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99

PUBLICITY ISSUED DESCRIPTION

N 44 45

8307250467 830711 PDR ADOCK 05000280 S PDR

NRC USE ONLY

NAME OF PREPARED J. L. Wilson

PHONE (804) 357-3184

ATTACHMENT 1

SURRY POWER STATION, UNIT NO. 1

DOCKET NO: 50-280

REPORT NO: 83-026/03L-0

EVENT DATE: 06-16-83

TITLE OF THE EVENT: CONTAINMENT PARTIAL PRESSURE ABOVE T.S. FIG. 3.8.1

1. Description of the Event

On June 16, with Unit 1 reactor critical at 0% power, the containment partial air pressure was at a value greater than that stipulated in T.S. Fig. 3.8.1. This event occurred again on June 17, with the unit at 35% power. These events are contrary to T.S.-3.8.A.4 and reportable per T.S. 6.6.2.b.(2).

2. Probable Consequences and Status of Redundant Equipment

At no time during the event was the actual containment partial pressure greater than .25 psi above T.S. Fig. 3.8.1. At this value, the CLS systems are capable of maintaining containment pressure below 45 psig and depressurize the containment to 0.0 psig in less than one hour following a LOCA. Therefore, the health and safety of the public would not have been affected.

3. Cause

One of the parameters affecting allowable containment partial pressure is service water temperature. At the time of the event, actual containment pressure was decreasing slightly, however service water temperature was increasing rapidly. As a result, the value stipulated in T.S. Fig. 3.8.1, was lowered to a point below the actual containment pressure.

4. Immediate Corrective Action

Containment pressure continued to be decreased to a point below the value in T.S. Fig. 3.8.1.

5. Subsequent Corrective Action

None.

6. Action Taken to Prevent Recurrence

None deemed necessary.

7. Generic Implications

None.

Vepco

USNRC REGION II
ATLANTA, GEORGIA

83 JUL 19 A10:30

VIRGINIA ELECTRIC AND POWER COMPANY
Surry Power Station
P. O. Box 315
Surry, Virginia 23883

Serial No: 83-049

Docket No: 50-280

License No: DPR-32

JUL 11 1983

Mr. James P. O'Reilly
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly

Pursuant to Surry Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 1.

Report Number

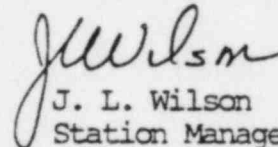
83-026/03L-0

Applicable Technical Specification

T. S. 6.6.2.b(2)

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,


J. L. Wilson
Station Manager

Enclosure

cc: Document Control Desk, USNRC
016 Phillips Bldg.
Washington, D. C. 20555

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