

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 C A S O S 2 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40

CON'T

01 REPORT SOURCE 05 00 03 6 1 7 0 6 0 9 8 3 8 0 7 1 1 8 3 9  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 With Unit 2 in Mode 2, Control Element Assembly (CEA) #57 (Regulating Group 4)

03 slipped more than 19 inches during group adjustment. The CEA was re-aligned per

04 Procedure S023-3-5.8 at 1415 on 6/9/83. Five minutes later CEA #57 slipped more

05 than 19 inches again and was re-aligned per the same procedure at 1435 on

06 6/9/83. In both cases Technical Specification 3.1.3.1 Action Statement 'c'

07 was entered and satisfied within the one hour time limit. Public health and

08 safety were not affected by this event.

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE  
R B 11 B 12 A 13 C R D R V E 14 Z 15 Z 16

17 LER/RO REPORT NUMBER 8 3 0 6 5 0 3 L 0

18 ACTION TAKEN 19 FUTURE ACTION 20 EFFECT ON PLANT 21 SHUTDOWN METHOD 22 HOURS 23 ATTACHMENT SUBMITTED 24 NPD-4 FORM SUB. 25 PRIME COMP. SUPPLIER 26 COMPONENT MANUFACTURER  
X 18 F 19 Z 20 Z 21 0 0 0 0 N 23 N 24 N 25 C 4 9 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The cause of this event was attributed to malfunctioning of the upper gripper on

11 the CEA. As corrective action the CEA coil stacks will be replaced with more

12 powerful System 80 coils as described in LER 83-054 during the next refueling

13 outage. See also LER's 82-084, 82-085, 82-086, 82-088, 82-091, 82-092, and

14 82-128.

15 FACILITY STATUS 16 % POWER 17 OTHER STATUS 18 METHOD OF DISCOVERY 19 DISCOVERY DESCRIPTION  
B 28 0 0 1 29 NA A 31 Operator Observation 32

16 ACTIVITY CONTENT 17 RELEASED OF RELEASE 18 AMOUNT OF ACTIVITY 19 LOCATION OF RELEASE  
Z 33 Z 34 NA NA 36

17 PERSONNEL EXPOSURES 18 NUMBER 19 TYPE 20 DESCRIPTION 21  
0 0 0 17 Z 20 NA 39

18 PERSONNEL INJURIES 19 NUMBER 20 DESCRIPTION 21  
0 0 0 40 NA 41

19 LOSS OF OR DAMAGE TO FACILITY 20 TYPE 21 DESCRIPTION 22  
Z 42 NA 43

20 PUBLICITY ISSUED 21 DESCRIPTION 22  
N 44 45

NAME OF PREPARER

NA H. B. RAY

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**SCE**

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H. B. RAY  
STATION MANAGER

July 11, 1983

U.S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region V  
1450 Maria Lane, Suite 210  
Walnut Creek, California 94596-5368

Attention: Mr. J.B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-361  
30-Day Report  
Licensee Event Report No. 83-065  
San Onofre Nuclear Generating Station, Unit 2

Pursuant to Section 6.9.1.13.b of Appendix A, Technical Specifications to Facility Operating License NPF-10 for San Onofre Unit 2, this submittal provides the required 30-day written report and a copy of the Licensee Event Report (LER) form for an occurrence involving Limiting Condition for Operation (LCO) 3.1.3.1 associated with the Reactivity Control System. Enclosed LER 83-065 addresses this event.

If there are any questions regarding the above, please contact me.

Sincerely,

*HE Moya*

CAMorris:1152u:dkd

Enclosure: LER No. 83-065

July 11, 1983

cc: A.E. Chaffee (USNRC Resident Inspector, Units 2 and 3)  
R.J. Pate (USNRC Resident Inspector, Units 2 and 3)  
J.P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U.S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement

U.S. Nuclear Regulatory Commission  
Office of Management Information and Program Control (MIPC)

Institute of Nuclear Power Operations (INPO)