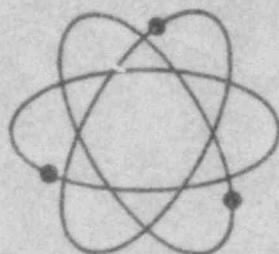


# **Vepco**

## **NORTH ANNA UNIT 2, CYCLE 3 STARTUP PHYSICS TEST REPORT**



**NUCLEAR OPERATIONS DEPARTMENT**

**Virginia Electric and Power Company**

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NORTH ANNA UNIT 2, CYCLE 3  
STARTUP PHYSICS TEST REPORT

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## PREFACE

The purpose of this report is to present the analysis and evaluation of the physics tests which were performed to verify that the North Anna 2, Cycle 3 core could be operated safely, and to make an initial evaluation of the performance of the core. It is not the intent of this report to discuss the particular methods of testing or to present the detailed data taken. Standard test techniques and methods of data analysis were used. The test data, results, and evaluations, together with the detailed startup procedures, are on file at the North Anna Power Station. Therefore, only a cursory discussion of these items is included in this report. The analyses presented includes a brief summary of each test, a comparison of the test results with design predictions, and an evaluation of the results.

The North Anna 2, Cycle 3 Startup Physics Tests Results and Evaluation Sheets have been included as an appendix to provide additional information on the startup test results. Each data sheet provides the following information: 1) test identification, 2) test conditions (design), 3) test conditions (actual), 4) test results, 5) acceptance criteria, and 6) comments concerning the test. These sheets provide a compact summary of the startup test results in a consistent format. The design test conditions and design values of the measured parameters were completed prior to startup physics testing. The entries for the design values were based on the calculations performed by Vepco's Nuclear Fuel Engineering Group<sup>1</sup>. During the tests, the data sheets were used as guidelines both to verify that the proper test conditions were met and to facilitate the preliminary comparison between measured and predicted test results, thus enabling a quick identification of possible problems occurring during the tests. The Appendix to this report contains the final completed and approved version of the Startup Physics Tests Results and Evaluation Sheets.

## Section 1

### INTRODUCTION AND SUMMARY

On April 2, 1983, Unit No. 2 of the North Anna Power Station was shut down for its second refueling. During this shutdown, 56 of the 157 fuel assemblies in the core were replaced with fresh fuel assemblies. The core loading pattern and the design parameters for each batch are shown in Figure 1.1. Fuel assembly burnups are given in Figure 1.2. The incore instrumentation locations are identified in Figure 1.3. Figure 1.4 identifies the the location and number of burnable poison rods in the Cycle 3 core. Figure 1.5 identifies the location and number of control rods in the Cycle 3 core.

On May 27, 1983, at 0629, the third cycle core achieved initial criticality. Following criticality, startup physics tests were performed as outlined in Table 1.1. A summary of the results of these tests follows:

1. The drop time of each control rod was confirmed to be within the 2.2 second limit of the North Anna Technical Specifications<sup>2</sup>.
2. Individual control rod bank worths for all control rod banks were measured using the rod swap technique<sup>3</sup> and were found to be within 13.6% of the design predictions. The sum of the individual control rod bank worths was measured to be within 2.6% of the design prediction. These results are within the design tolerance of  $\pm 15\%$  for individual bank worths ( $\pm 10\%$  for the rod swap reference bank worth) and the design tolerance of  $\pm 10\%$  for the sum of the individual control rod bank worths.
3. Critical boron concentrations for two control bank configurations were measured to be within 29 ppm of the

design predictions. These results were within the design tolerances and also met the accident analysis acceptance criterion.

4. The boron worth coefficient was measured to be within 1.1% of the design prediction, which is well within the design tolerance of  $\pm 10\%$  and met the accident analysis criterion.
5. The isothermal temperature coefficient was measured to be within 0.05 pcm/ $^{\circ}\text{F}$  of design prediction. This result is within the design tolerance of  $\pm 3$  pcm/ $^{\circ}\text{F}$  and also met the accident analysis acceptance criterion.
6. Core power distributions for various HZP and at power conditions were generally within 5% of the predicted power distributions. For all maps, the hot channel factors were measured to be within the limits of the Technical Specifications. However, at power levels less than 79%, there were violations of the Radial Peaking Factor,  $F_{xy}(\text{RTP})$ , surveillance limit, and at 4% power, a quadrant power tilt ratio of 2.76% was measured. Generally, all measurement parameters met their respective design value tolerances. All measurement parameters met their respective accident analysis acceptance criteria.

In summary, all startup physics test results were acceptable. Detailed results, together with specific design tolerances and acceptance criteria for each measurement, are presented in the appropriate sections of this report.



Table 1.1

NORTH ANNA 2 - BOL CYCLE 3 PHYSICS TESTS  
CHRONOLOGY OF TESTS

Test	Date	Time	Power	Reference Procedure
Hot Rod Drop-Hot Full Flow	5/26/83	1243	HSD	2-PT-17.2
Reactivity Computer Checkout	5/27/83	1338	HZP	2-PT-94.2
Boron Endpoint-ARO	5/28/83	0640	HZP	2-PT-94.3
Temperature Coefficient-ARO	5/28/83	0746	HZP	2-PT-94.4
Bank B Worth	5/28/83	1139	HZP	2-PT-94.5
Boron Endpoint-B In	5/28/83	1655	HZP	2-PT-94.3
Bank D Worth - Rod Swap	5/28/83	1737	HZP	2-PT-94.7
Bank C Worth - Rod Swap	5/28/83	1837	HZP	2-PT-94.7
Bank A Worth - Rod Swap	5/28/83	1927	HZP	2-PT-94.7
Bank SB Worth - Rod Swap	5/28/83	2011	HZP	2-PT-94.7
Bank SA Worth - Rod Swap	5/28/83	2056	HZP	2-PT-94.7
Flux Map - ARO	5/29/83	0512	4%	2-PT-21.1
Flux Map - Verify Fxy	5/29/83	1840	31%	2-PT-21.1
Flux Map - I/E Calibration	6/06/83	0823	50%	2-PT-21.1
Flux Map - I/E Calibration	6/07/83	0606	79%	2-PT-21.1
Flux Map - Delta I Target	6/08/83	0806	97%	2-PT-21.1
Flux Map - Delta I Target	6/08/83	1632	99%	2-PT-21.1
Flux Map - I/E Calibration	6/09/83	1057	100%	2-PT-21.1
Flux Map - HZP, Eq. Xenon	6/10/83	0749	100%	2-PT-21.1

NORTH ANNA UNIT 2 - CYCLE 3  
CORE LOADING MAP



	3A2	4A	5A
Initial Enrichment (w/o U235)	3.10	3.41	3.59
Burnup at BOC-3 (MWD/MTU)	20,319	7,763	0
Assembly Type	17x17	17x17	17x17
Number of Assemblies	49	52	56
Fuel Rods Per Assembly	264	264	264

Figure 1.2

NORTH ANNA UNIT 2 - CYCLE 3  
BEGINNING OF CYCLE FUEL ASSEMBLY BURNUPS

R P M M L K J H G F E D C B A															
						P16	R07	P34							1
						23504	6088	23629							2
				R02	S22	S34	P30	S36	307	R03					3
				9238	0	0	18354	0	0	9198					4
			R15	R50	S54	P17	R49	P45	S46	R17	R24				5
			6517	8534	0	21962	7569	21742	0	8520	6705				6
		R33	R37	S27	R34	S33	P23	S19	R22	S32	R28	R05			7
		6457	6255	0	9675	0	18527	0	9977	0	6201	6625			8
	R11	R47	S20	P06	S25	P51	R27	P27	S30	P33	S05	R10	R38		9
	9019	8428	0	23277	0	18789	6184	18319	0	23210	0	8603	9306		10
	S44	S04	R45	S16	P31	S53	P05	S21	P10	S23	R39	S10	S09		11
	0	0	9698	0	20255	0	17930	0	18586	0	9662	0	0		12
P47	S24	P08	S12	P44	S45	P32	R19	P28	S41	P52	S47	P19	S31	P15	13
23458	0	22083	0	18546	0	17000	6412	16686	0	20096	0	21792	0	23403	14
R26	P37	R51	P20	R13	P43	R40	P50	R30	P49	R23	P24	R04	P38	R14	15
6118	18467	7F91	18277	5984	17899	6264	23614	6134	17920	6214	18450	7503	18497	6054	
P01	S41	P36	S42	P13	S03	P14	R36	P25	S14	P48	S29	P07	S02	P35	
23481	0	21663	0	18546	0	18863	6050	16962	0	18484	0	21716	0	23688	
	S13	S49	R35	S17	P41	S40	P02	S38	P22	S55	R06	S52	S48		
	0	0	9753	0	19931	0	18127	0	20113	0	9926	0	0		
	R46	R48	S43	P40	S18	P12	R18	P42	S26	P26	S08	R03	R44		
	9332	8636	0	23325	0	18637	6411	18543	0	23476	0	8822	9296		
	R16	R25	S28	R42	S39	P29	S50	R41	S51	R20	R29				
	6639	6198	0	9569	0	18509	0	9720	0	6163	6662				
		R01	R43	S37	P03	R31	P21	S35	R12	R21					
		6728	9088	0	21976	7450	21770	0	8725	6625					
			R52	S01	S56	P11	S06	S15	R32						
			9711	0	0	18615	0	0	9360						
						P09	R09	P18							
						23164	6031	23782							

---> ASSEMBLY ID

---> ASSEMBLY BURNUP

Figure 1.3

NORTH ANNA UNIT 2 - CYCLE 3  
INCORE INSTRUMENTATION LOCATIONS

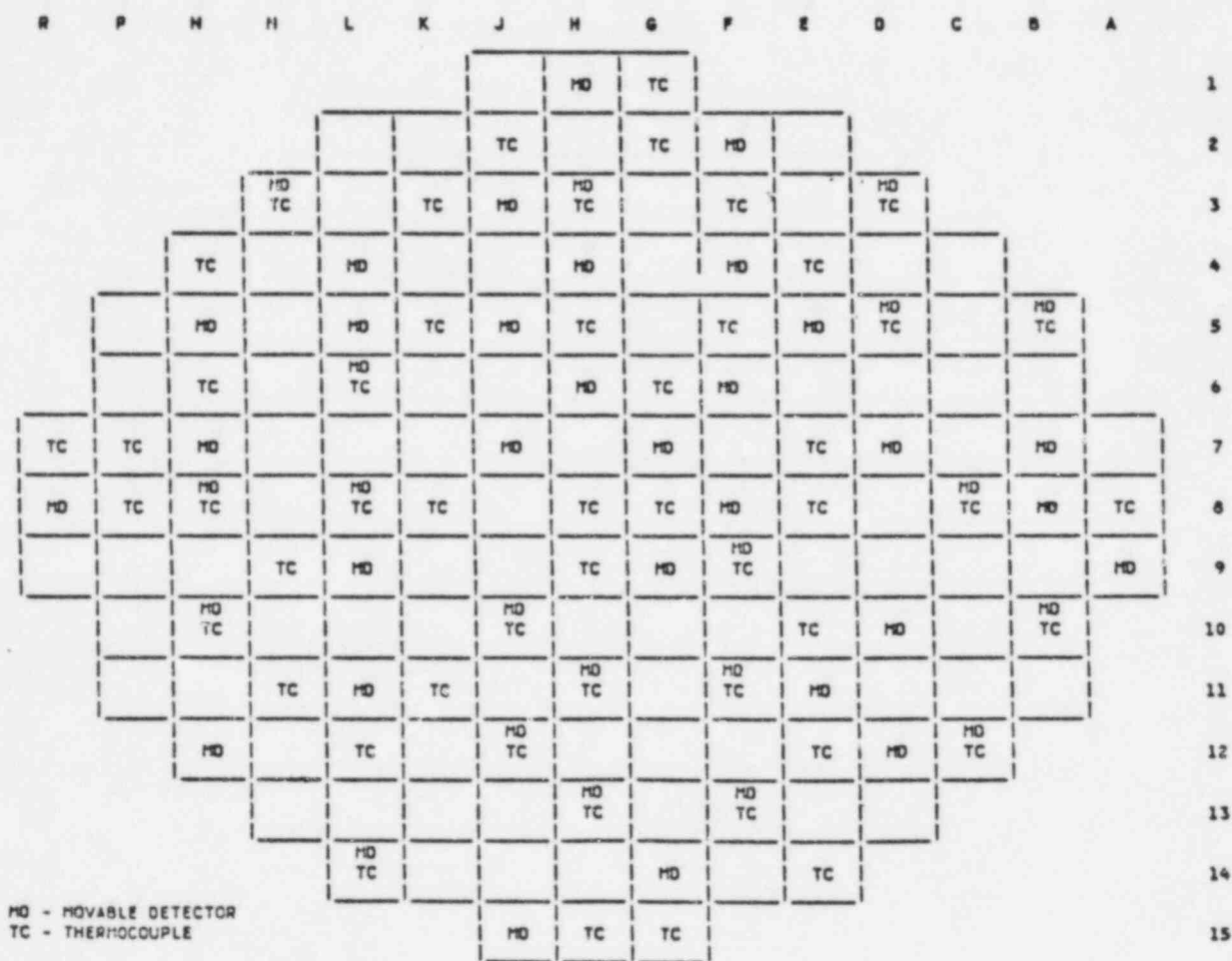


Figure 1.4

NORTH ANNA UNIT 2 - CYCLE 3  
BURNABLE POISON AND SOURCE ASSEMBLY LOCATIONS

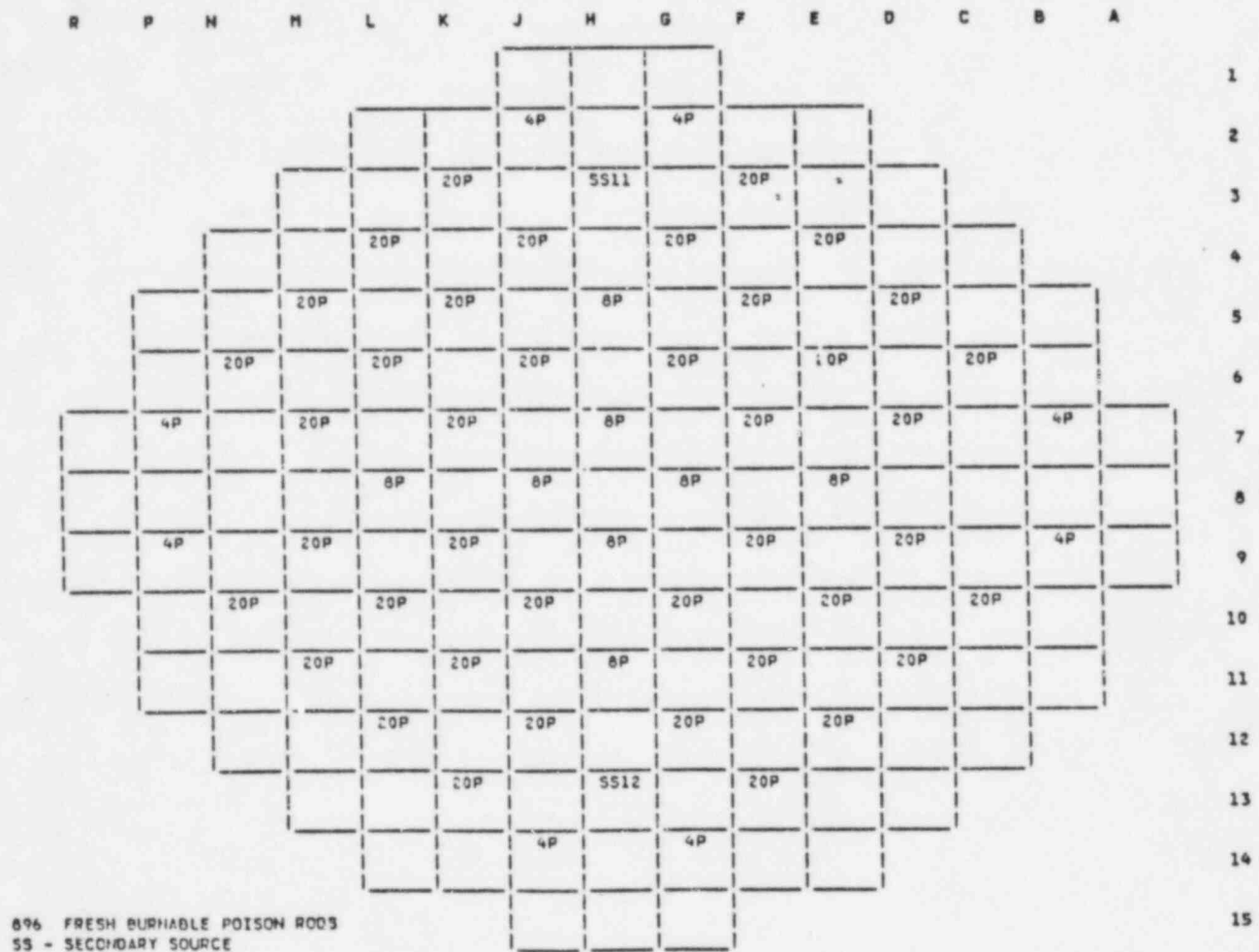
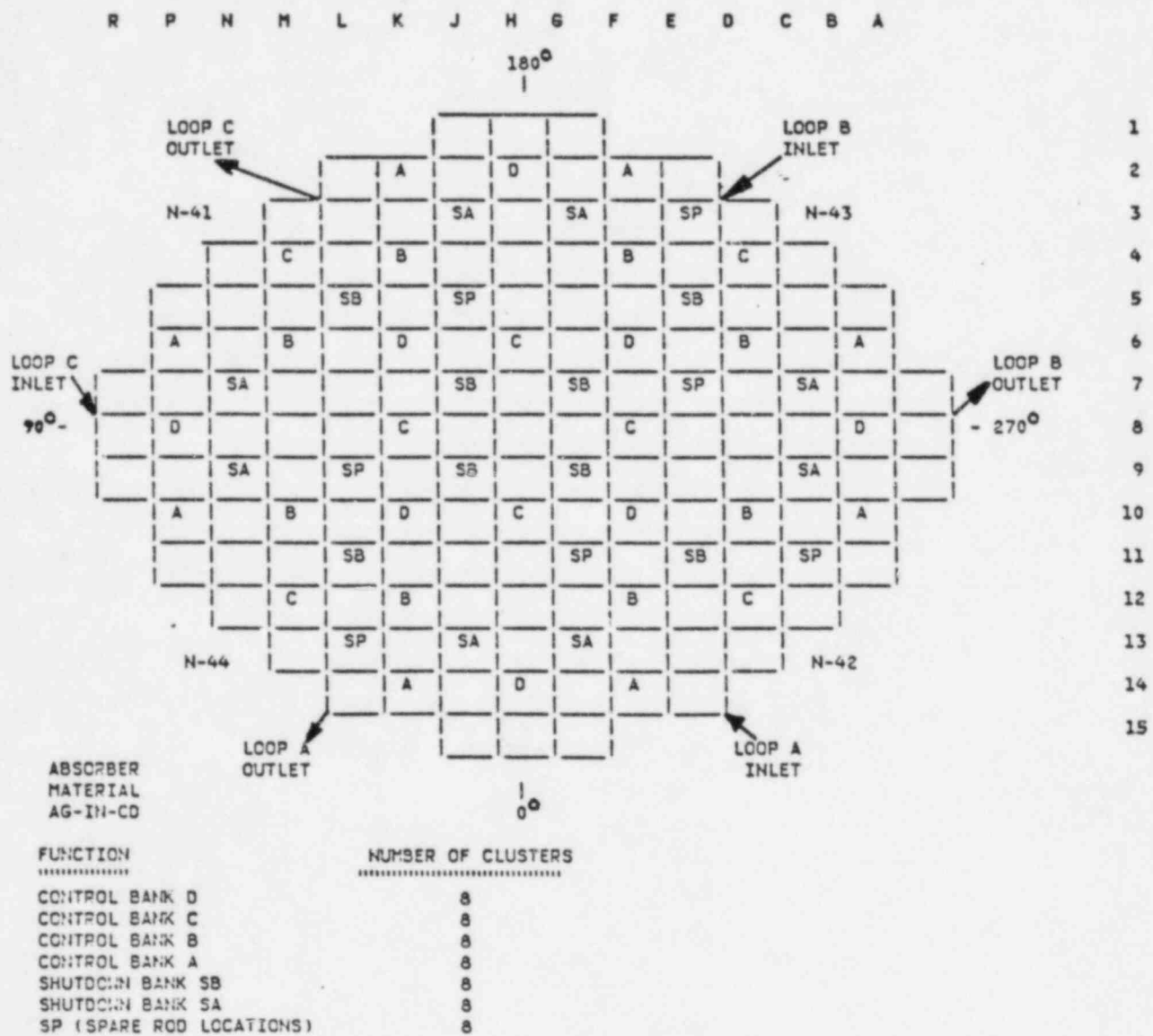


Figure 1.5

NORTH ANNA UNIT 2 - CYCLE 3  
CONTROL ROD LOCATIONS





## Section 2

### CONTROL ROD DROP TIME MEASUREMENTS

The drop time of each control rod was measured at hot RCS conditions in order to confirm satisfactory operation and to verify that the rod drop times were less than the maximum allowed by the Technical Specifications. The hot control rod drop time measurements were run with the RCS at hot, full flow conditions ( 547 °F, 2235 psig) and are described below.

The rod drop time measurements were performed by first withdrawing a rod bank to its fully withdrawn position, and then removing the movable gripper coil fuse and stationary gripper coil fuse for the test rod. This allows the rod to drop into the core as it would in a normal plant trip. The data recorded during this test are, the stationary gripper coil voltage, the LVDT (Linear Variable Differential Transformer) primary coil voltage and a 60Hz timing trace which are recorded via a visicorder. The rod drop time to the dashpot entry and to the bottom of the dashpot are determined from this data. Figure 2.1 provides an example of the data that is recorded during a rod drop time measurement.

As shown in Figure 2.1, the initiation of the rod drop is indicated by the decay of the stationary gripper coil voltage when the stationary coil fuse is removed. A voltage is then induced in the LVDT primary coil as the rod drops. The magnitude of this voltage is a function of the rod velocity. When the rod enters the dashpot section of its guide tube, the velocity slows causing a voltage decrease in the LVDT coil. The LVDT voltage then reaches a minimum as the rod reaches the bottom of the dashpot. Subsequent variations in the trace are caused by the rod

bouncing. This procedure was repeated for each control rod.

The measured drop times for each control rod are recorded on Figure 2.2. The slowest, fastest, and average drop times are summarized in Table 2.1. Technical Specification 3.1.3.4 specifies a maximum rod drop time from loss of stationary gripper coil voltage to dashpot entry of 2.2 seconds with the RCS at hot, full flow conditions. All test results met this limit.

Table 2.1

NORTH ANNA UNIT 2 - CYCLE 3 BOL PHYSICS TEST  
HOT ROD DROP TIME SUMMARY

ROD DROP TIME TO DASHPOT ENTRY

SLOWEST ROD	FASTEST ROD	AVERAGE TIME
B-10, 1.82 Sec.	M-4, 1.43 Sec.	1.61 Sec.

ROD DROP TIME TO BOTTOM OF DASHPOT

SLOWEST ROD	FASTEST ROD	AVERAGE TIME
B-6, 2.49 Sec.	M-4, 2.06 Sec.	2.29 Sec.

Figure 2.1

NORTH ANNA UNIT 2 - CYCLE 3 BOL PHYSICS TEST

TYPICAL ROD DROP TRACE

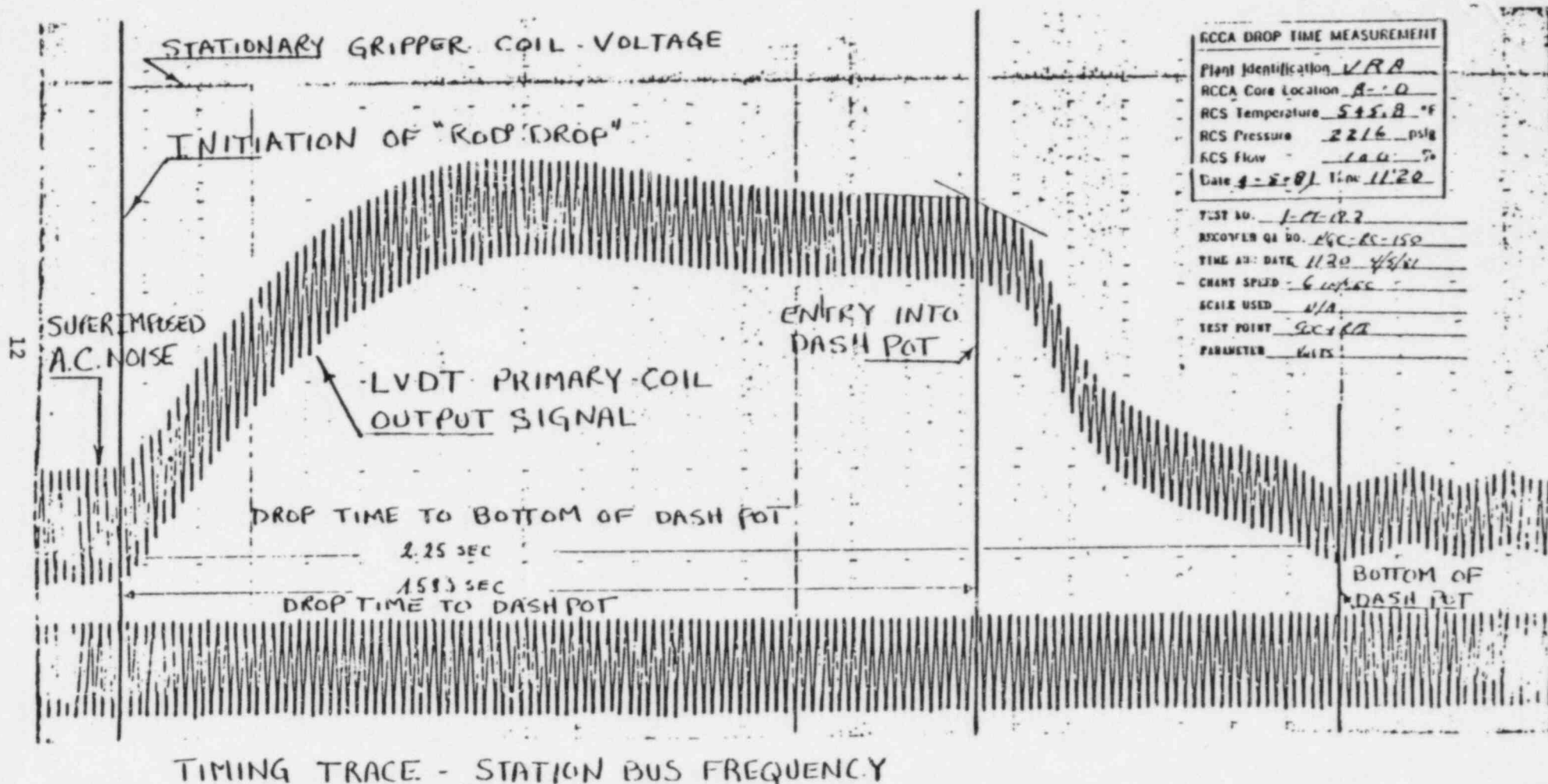
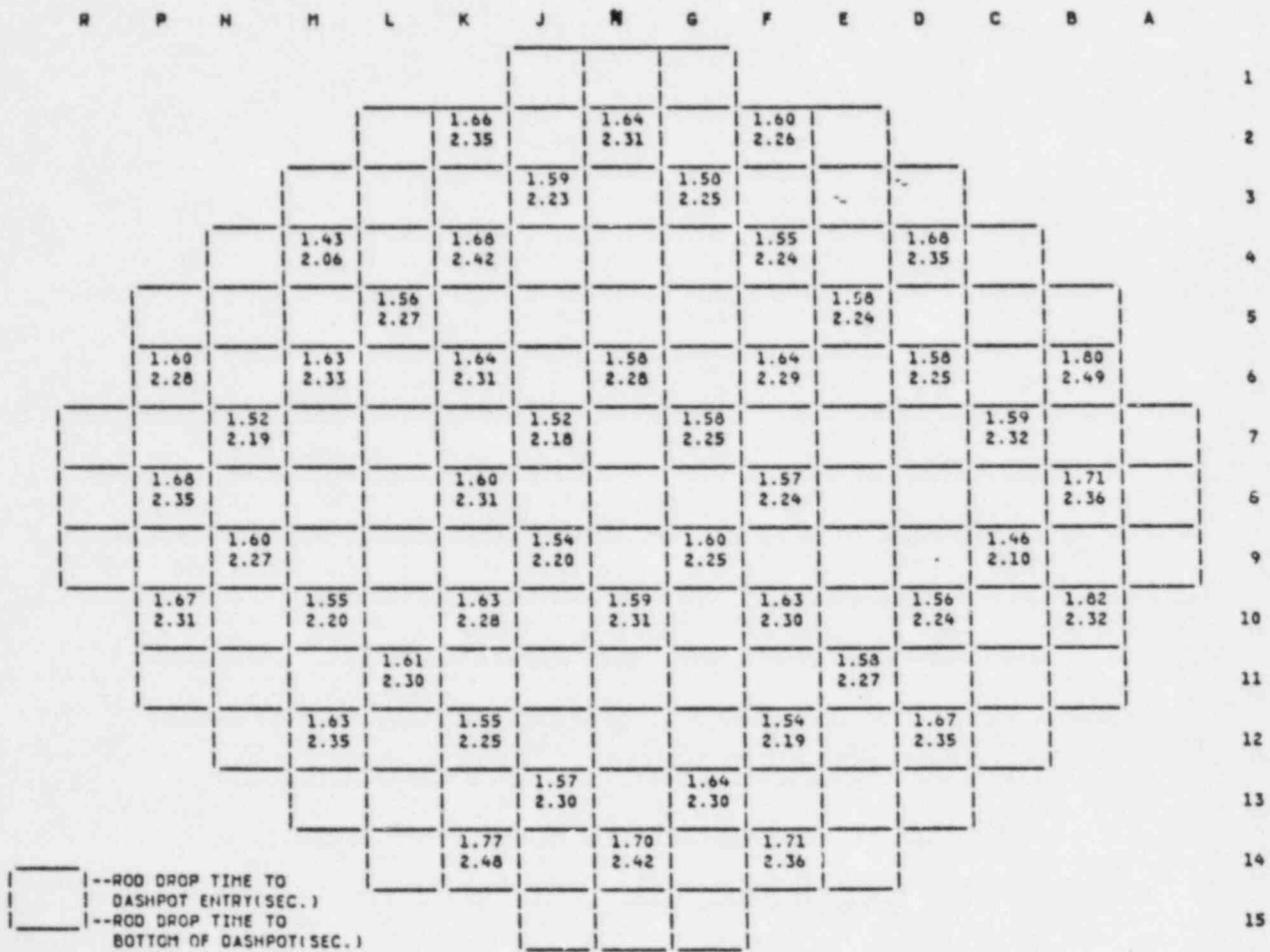


Figure 2.2

NORTH ANNA UNIT 2 - CYCLE 3 BOL PHYSICS TEST  
ROD DROP TIMES - HOT FULL FLOW CONDITIONS



### Section 3

#### CONTROL ROD BANK WORTH MEASUREMENTS

Control rod bank worth measurements were obtained for all control and shutdown banks using the rod swap technique. The first step in the rod swap procedure was to dilute the most reactive control rod bank (hereafter referred to as the reference bank) into the core and measure its reactivity worth using conventional test techniques. The reactivity changes resulting from the reference bank movements were recorded continuously by the reactivity computer<sup>5</sup> and were used to determine the differential and integral worth of the reference bank (Control Bank B). At the completion of the reference bank reactivity worth measurement, the reactor coolant system temperature and boron concentration were stabilized such that the reactor was critical with the reference bank near full insertion. Initial statepoint data for the rod swap maneuver were obtained by moving the reference bank to its fully inserted position and recording the core reactivity and moderator temperature. At this point, a rod swap maneuver was performed by withdrawing the reference bank while one of the other control rod banks (i.e., a test bank) was inserted. The core was kept nominally critical throughout this rod swap and the maneuver was continued until the test bank was fully inserted and the reference bank was at the position at which the core was just critical. This measured critical position (MCP) of the reference bank with the test bank fully inserted is the major parameter of interest and was used to determine the integral reactivity worth of the test bank. Statepoint data (core reactivity, moderator temperature, and the differential worth of the reference bank) were recorded with the reference bank at the MCP. The rod swap maneuver was then performed in reverse order such that the reference bank once again was near full insertion and the test bank was



once again fully withdrawn from the core. The rod swap process was then repeated for all of the other control rod banks (control and shutdown).

A summary of the results for these tests is given in Table 3.1. As shown by this table and the Startup Physics Test Results and Evaluation Sheets given in the Appendix, the individual measured bank worths for all of the control and shutdown banks were within the design tolerance ( $\pm 10\%$  for the reference bank and  $\pm 15\%$  for the test banks). The sum of the individual rod bank worths was measured to be within 2.6% of the design prediction. This is well within the design tolerance of  $\pm 10\%$  for the sum of the individual control rod bank worths.

The integral and differential reactivity worths of the reference bank (Control Bank B) are shown in Figures 3.1 and 3.2, respectively. The design predictions and the measured data are plotted together in order to illustrate their agreement. In summary, all measured rod worth values were satisfactory.

Table 3.1

NORTH ANNA UNIT 2 - CYCLE 3 BOL PHYSICS TEST  
CONTROL ROD BANK WORTH SUMMARY

BANK	MEASURED WORTH (PCM)	PREDICTED WORTH (PCM)	PERCENT DIFFERENCE $(M-P)/P \times 100$
B-Reference Bank	1569	1544	1.6
D	859	850	1.1
C	896	903	-0.8
A	758	667	13.6
SB	806	827	-2.5
SA	995	945	5.3
Total Worth	5883	5736	2.6

Figure 3.1  
NORTH ANNA UNIT 2 - CYCLE 3 BOL PHYSICS TEST  
BANK B INTEGRAL ROD WORTH - HZP  
BANK B WITH ALL OTHER RODS OUT

-- PREDICTED

\* MEASURED

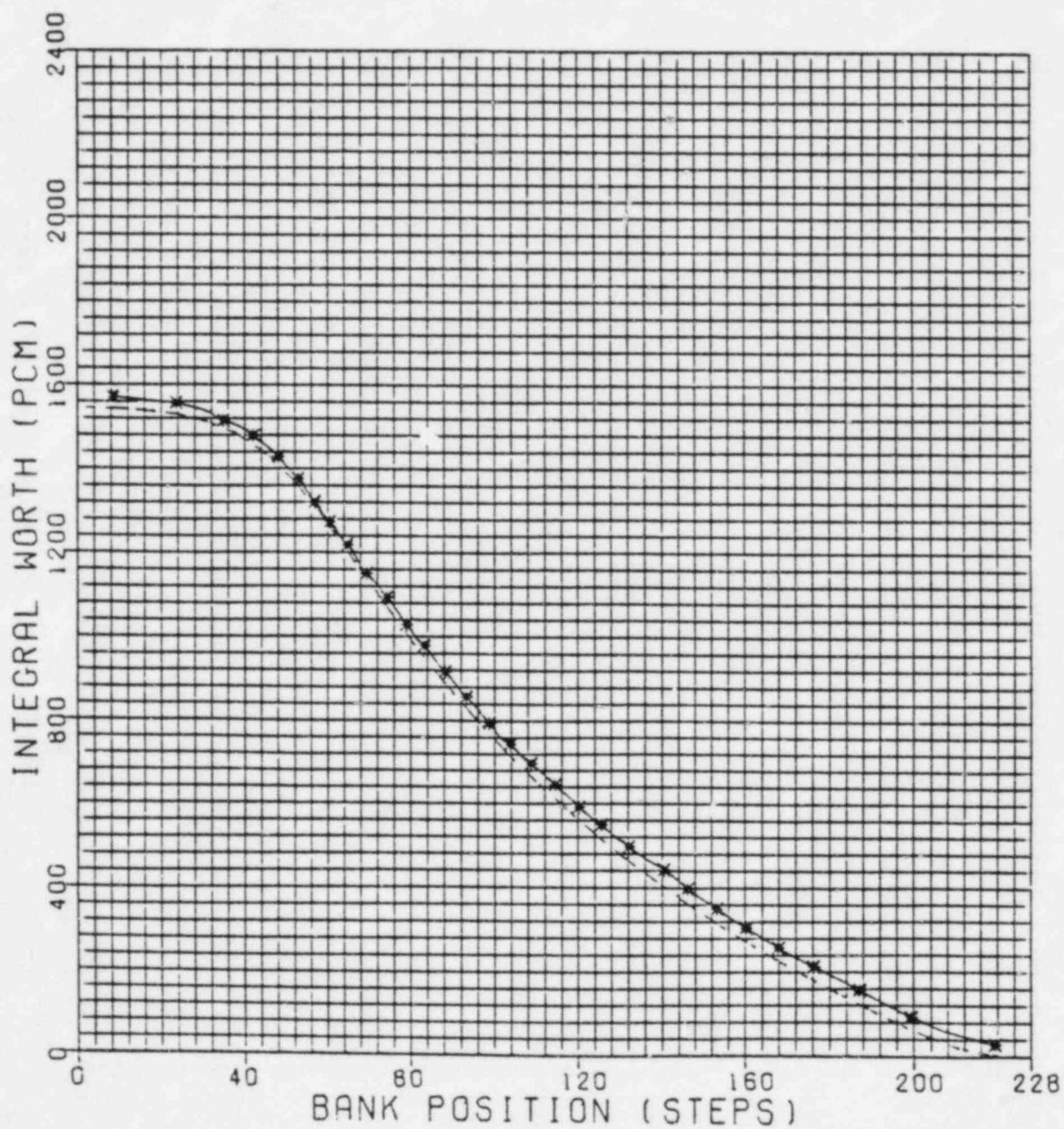
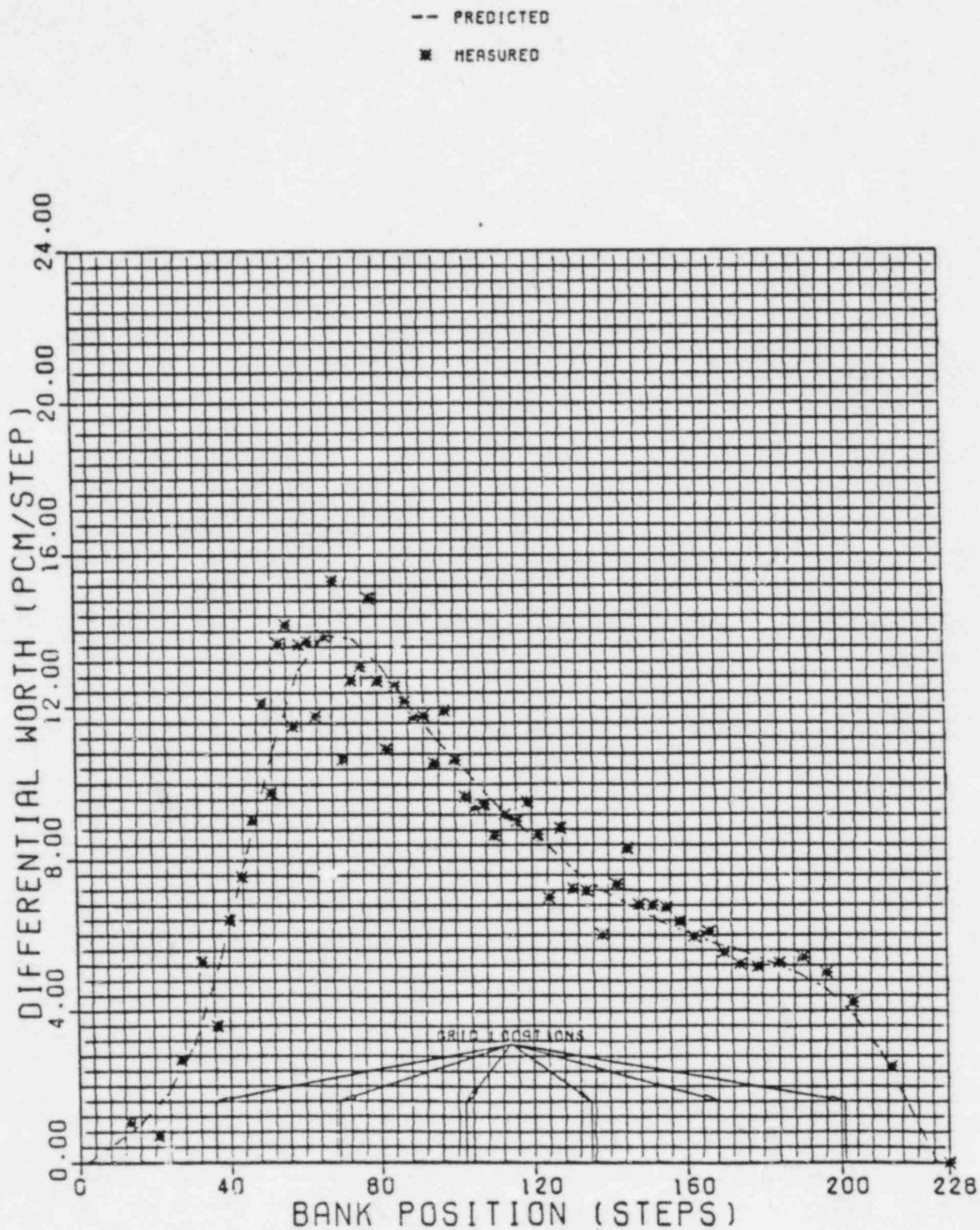


Figure 3.2  
 NORTH ANNA UNIT 2 - CYCLE 3 BOL PHYSICS TEST  
 BANK B DIFFERENTIAL ROD WORTH - HZP  
 BANK B WITH ALL OTHER RODS OUT



## Section 4

### BORON ENDPOINT AND WORTH MEASUREMENTS

#### Boron Endpoint

With the reactor critical at hot zero power, reactor coolant system boron concentrations were measured at selected rod bank configurations to enable a direct comparison of measured boron endpoints with design predictions. For each measurement, the RCS conditions were stabilized with the control banks at or very near a selected endpoint position. The critical boron concentration was then measured. If necessary, an adjustment to the measured critical boron concentration was made to account for off-nominal core conditions, such as rod position and moderator temperature.

The results of these measurements are given in Table 4.1. As shown in this table and in the Startup Physics Test Results and Evaluation Sheets given in the Appendix, all measured critical boron endpoint values were within their respective design tolerances. All measured values met the accident analysis acceptance criterion. In summary, all results were satisfactory.

#### Boron Worth Coefficient

The measured boron endpoint values provide stable statepoint data from which the boron worth coefficient was determined. A plot of the boron concentration as a function of integrated reactivity can be constructed by relating each endpoint concentration to the integrated rod worth present in the core at the time of the endpoint measurement. The value of the boron coefficient, over the range of boron endpoint concentrations, is

obtained directly from this plot.

The boron worth plot is shown in Figure 4.1. As indicated in this figure and in the Appendix, the boron worth coefficient of reactivity was measured to be -7.85 pcm/ppm. The measured boron worth coefficient is within -1.1% of the predicted value of -7.94 pcm/ppm and is well within the design tolerance of  $\pm 10\%$ . The measurement result also met the accident analysis acceptance criterion. In summary, this result was satisfactory.



Table 4.1

NORTH ANNA UNIT 2 - CYCLE 3 BOL PHYSICS TEST  
BORON ENDPOINTS SUMMARY

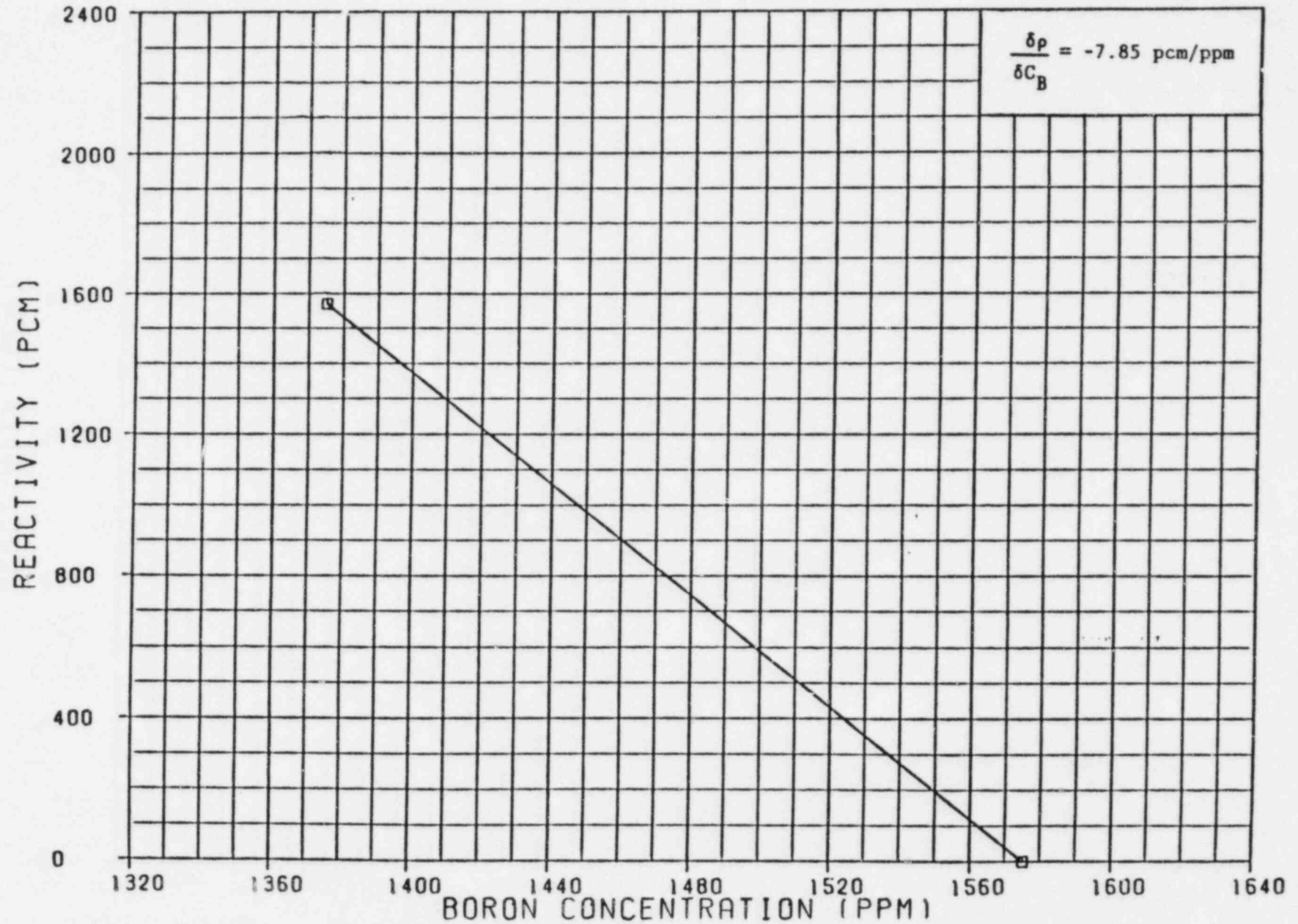
Control Rod Configuration	Measured Endpoint (ppm)	Predicted Endpoint (ppm)	Difference M-P (ppm)
ARO	1575	1546	29
B Bank In	1375	1387*	-12

\*The predicted endpoint for the B Bank in configuration has been adjusted for the difference between the measured and predicted values of the endpoint taken at the ARO configuration as shown in the boron endpoint Startup Physics Test Results and Evaluation Sheets in the Appendix.

Figure 4.1

NORTH ANNA 2 - CYCLE 3 BOL PHYSICS TEST  
BORON WORTH COEFFICIENT

□ ENDPOINT MEASUREMENTS



## Section 5

### TEMPERATURE COEFFICIENT MEASUREMENTS

The isothermal temperature coefficient measurements were accomplished by controlling the RCS heat gains/losses with the steam dump valves to the condenser, establishing a constant and uniform heatup/cooldown rate, and then monitoring the resulting reactivity changes on the reactivity computer. These measurements were performed at very low power levels in order to minimize the effects of non-uniform nuclear heating, thus, the moderator and fuel were approximately at the same temperature (between 544 and 548 °F) during these measurements. To eliminate the boron reactivity effect of outflow from the pressurizer, the pressurizer level was maintained constant or slightly increasing during these measurements.

Reactivity measurements were taken during both RCS heatup and cooldown ramps during which the RCS temperature varied approximately 4°F. Reactivity was determined using the reactivity stripchart recorder and temperature was recorded on the average RCS temperature stripchart. The temperature coefficient was then determined from the change in these parameters. The change in reactivity divided by the change in temperature yields the isothermal temperature coefficient.

The predicted and measured isothermal temperature coefficient values are compared in Table 5.1. As can be seen from this summary and from the Startup Physics Test Results and Evaluation Sheet given in the Appendix, the measured isothermal temperature coefficient value was within the design tolerance of  $\pm 3$  pcm/°F and met the accident analysis acceptance criterion. In summary, the measured result was satisfactory.

Table 5.1

NORTH ANNA UNIT 2 - CYCLE 3 BOL PHYSICS TESTS  
ISOTHERMAL TEMPERATURE COEFFICIENT SUMMARY

BANK POSITION	TEMPERATURE RANGE (°F)	BORON CONCENTRATION (PPM)	ISOTHERMAL TEMPERATURE COEFFICIENT (PCM/°F)				
			HEATUP	COOL DOWN	AVER.	PRED.	DIFFER (M-P)
All Rods Out	544.3 to 548.1	1575	-3.73	-3.56	-3.65	-3.70	0.05

## Section 6

### POWER DISTRIBUTION MEASUREMENTS

The core power distributions were measured using the incore movable detector flux mapping system. This system consists of five fission detectors which traverse fuel assembly instrumentation thimbles in 50 core locations (see Figure 1.3). For each traverse, the detector output is continuously monitored on a strip chart recorder. The output is also scanned for 61 discrete axial points by the PRODAC P-250 process computer. Full core, three-dimensional power distributions are then determined by analyzing this data using the Westinghouse computer program, INCORE<sup>6</sup>. INCORE couples the measured flux map data with predetermined analytic power-to-flux ratios in order to determine the power distribution for the whole core.

A list of all the flux maps taken during the test program together with a list of the measured values of the important power distribution parameters is given in Table 6.1. The measured power distribution parameter values are compared with their Technical Specifications limits in Table 6.2. Flux map 1 was taken at 4% power. This flux map served as the base case design check. Figure 6.1 shows the resulting radial power distribution associated with the flux map. Map 1 indicated the presence of a small quadrant power tilt violation (measured 2.76%) and violations of the rated thermal power limit for the radial peaking factor ( $F_{xy}(RTP)$ ).

Several additional flux maps were taken during power escalation for Fxy monitoring. A typical power distribution for one of these maps is shown in Figure 6.2. The measured values for Fxy continued to exceed Fxy(RTP) during the power escalation, until a map was taken at 79% power. No further violations of Fxy(RTP) were measured at higher power levels, and at no time was Fxy(L) exceeded.

Flux maps 3 through 10 were taken over a wide range of power levels and control rod configurations. These flux maps were taken to check the at-power design predictions and hot channel factors, establish a target delta flux for operation, and to measure core power distributions at various operating conditions. These maps also provide incore/excore detector calibration data for the nuclear instrumentation system. The radial power distributions for these maps are given in Figures 6.3 through 6.8. These figures show that the measured relative assembly power values are generally within 5% of the predicted values. Although the assemblywise relative power for Map 3, taken at 50% power, was slightly higher than the design prediction, the measured F-Q(T) and F-ΔH(N) peaking factor values for the at-power flux maps were within the Technical Specifications limits.

In conclusion, all power distribution measurement results were considered to be acceptable with respect to the design tolerances, the accident analysis acceptance criteria, and the Technical Specification limits. It is therefore anticipated that the core will continue to operate safely throughout Cycle 3.

Table 6.1

## NORTH ANNA UNIT 2 - CYCLE 3 BOL PHYSICS TESTS

## INCORE FLUX MAP SUMMARY

MAP DESCRIPTION	MAP NO.	DATE	BURN UP MBD/ MTU	PWR (%)	BANK D STEPS	1 F-Q(T) HOT CHANNEL FACTOR				2 F-DH(N) HOT CHNL.FACTOR			3 CORE F(Z) MAX		4 F(XY) MAX		QPTR MAX LOC	AXIAL OFF SET (%)	NO. OF THIN BLES
						ASSY	PIN	AXIAL POINT	F-Q(T)	ASSY	PIN	F-DH(N)	AXIAL POINT	F(Z)	F(XY) MAX	MAX			
ARD	1	5-29-83	0	4	211	M10	ED	20	2.479	M10	ED	1.574	21	1.514	1.662	1.028	NH	22.96	41
VERIFY FXY (5)	2	5-29-83	0	31	177	K14	MH	29	2.279	K14	MH	1.570	29	1.386	1.646	1.016	SW	0.60	41
VERIFY FXY	3	6- 6-83	85	50	179	K14	MH	29	2.139	K14	MH	1.545	29	1.320	1.620	1.015	SW	-1.82	45
VERIFY FXY (6)	6	6- 7-83	112	79	192	K14	MH	28	1.980	K14	MH	1.480	29	1.270	1.555	1.011	SW	-0.30	50
DELTA I TARGET	7	6- 8-83	140	97	228	K14	MH	37	1.915	K14	MH	1.474	37	1.241	1.557	1.008	SW	-3.85	46
DELTA I TARGET	8	6- 8-83	155	99	228	K14	MH	37	1.940	K14	MH	1.475	37	1.255	1.558	1.009	SW	-5.94	50
I/E CAL.	9	6- 8-83	185	100	210	K14	MH	38	1.976	K14	MH	1.476	37	1.275	1.555	1.010	SW	-7.36	49
HFP, EQ. XENON	10	6-10-83	215	100	228	K14	MH	29	1.906	K14	MH	1.473	37	1.229	1.558	1.009	SW	-2.10	49

NOTES: HOT SPOT LOCATIONS ARE SPECIFIED BY GIVING ASSEMBLY LOCATIONS (E.G. H-8 IS THE CENTER-OF-CORE ASSEMBLY), FOLLOWED BY THE PIN LOCATION (DENOTED BY THE "Y" COORDINATE WITH THE SEVENTEEN ROWS OF FUEL RODS LETTERED A THROUGH R AND THE "X" COORDINATE DESIGNATED IN A SIMILAR MANNER). IN THE "Z" DIRECTION THE CORE IS DIVIDED INTO 61 AXIALPOINTS STARTING FROM THE TOP OF THE CORE.

1. F-Q(T) INCLUDES A TOTAL UNCERTAINTY OF  $1.05 \times 1.03$
2. F-DH(N) INCLUDES A MEASUREMENT UNCERTAINTY OF 1.04
3. F(XY) INCLUDES A TOTAL UNCERTAINTY OF  $1.05 \times 1.03$ .
4. QPTR - QUADRANT POWER TILT RATIO.
5. FLUX MAPS 2, 3, AND 6 WERE TAKEN TO EXAMINE F-XY WITH RESPECT TO THE F-XY(RTP) SURVEILLANCE LIMIT.
6. FLUX MAPS 4 AND 5 WERE QUARTER-CORE MAPS TAKEN FOR PRELIMINARY EXCORE DETECTOR CALIBRATION.



Table 6.2

NORTH ANNA UNIT 2 - CYCLE 3 BOL PHYSICS TESTS  
COMPARISON OF MEASURED POWER DISTRIBUTION PARAMETERS  
WITH THEIR TECHNICAL SPECIFICATION LIMITS

MAP NO.	F-Q(T) HOT CHANNEL FACTOR <sup>1</sup>			F-DH(N) HOT CHANNEL FACTOR <sup>2</sup>			F(XY) MAX <sup>3</sup>			
	MEAS	LIMIT	MARGIN (%)	MEAS	LIMIT	MARGIN (%)	MEAS	AXIAL POINT	LIMIT	MARGIN (%)
2	2.28	4.38	47.9	1.57	1.87	16.0	1.65	35	1.82	9.3
3	2.14	4.38	51.1	1.55	1.78	12.9	1.62	29	1.76	8.0
6	1.98	2.77	28.5	1.48	1.65	10.3	1.56	29	1.67	6.6
7	1.92	2.27	15.4	1.47	1.56	5.8	1.56	29	1.61	3.1
8	1.94	2.22	12.6	1.48	1.55	4.5	1.56	29	1.60	2.5
9	1.98	2.20	10.0	1.48	1.55	4.5	1.56	29	1.60	2.5
10	1.91	2.19	12.8	1.47	1.55	5.2	1.56	29	1.60	2.5

1 The Technical Specification's limit for the heat flux hot channel factor, F-Q(T) is a function of core height. The value for F-Q(T) listed above is the maximum of F-Q(T) in the core. The Technical Specification's limit listed above is evaluated at the plane of maximum F-Q(T). The minimum margin values listed above are the minimum percent difference between the measured values of F-Q(T) and the Technical Specification's limit for each map. All measured F-Q(T) hot channel factors include 5% measurement uncertainty and 3% engineering uncertainty.

2 The measured values for the enthalpy rise hot channel factor, F-ΔH(N) includes 4% measurement uncertainty.

3 All measured F(XY) MAX values include 5% measurement uncertainty and 3% engineering uncertainty.

	P	H	M	E	K	J	H	S	F	D	B	A		
PREDICTED	0.31	0.47	0.31											
MEASURED	0.33	0.51	0.32										1	
PCT DIFFERENCE	7.4	7.3	4.7											
0.56	1.03	1.11	0.80	1.11	1.03	0.56							2	
0.61	1.06	1.14	0.82	1.14	1.05	0.57								
0.3	3.0	2.9	2.7	2.5	1.7	1.7								
0.64	1.09	1.25	0.98	1.24	0.98	1.25	1.09	0.64					3	
0.69	1.18	1.29	0.99	1.24	0.98	1.27	1.11	-0.63						
0.2	8.3	3.0	0.2	-0.1	-0.4	1.7	-1.7	-1.2						
0.64	1.16	1.29	1.37	1.24	1.05	1.24	1.37	1.29	1.16	0.64			4	
0.69	1.23	1.37	1.39	1.25	1.04	1.24	1.35	1.27	1.14	0.65				
7.2	6.3	6.3	1.3	-0.8	-0.9	-1.4	-1.8	-1.5	-1.5	0.7				
0.56	1.09	1.28	1.00	1.22	1.01	1.16	1.01	1.22	1.00	1.28	1.09	0.56	5	
0.59	1.15	1.33	1.02	1.22	0.98	1.13	0.98	1.19	0.98	1.26	1.09	0.57		
6.2	6.2	3.4	2.1	0.1	-3.0	-2.9	-2.5	-2.2	-1.8	-1.5	0.8	2.7		
1.02	1.25	1.37	1.22	0.92	1.09	0.92	1.09	0.92	1.22	1.37	1.25	1.02	6	
1.10	1.33	1.43	1.24	0.91	1.05	0.89	1.05	0.90	1.20	1.35	1.25	1.04		
7.0	7.0	4.4	1.7	-1.0	-3.9	-3.8	-3.4	-2.7	-1.8	-1.5	0.5	1.3		
0.31	1.11	0.98	1.26	1.01	1.10	0.92	1.03	0.92	1.10	1.01	1.26	0.98	1.11	0.31
0.33	1.18	1.04	1.29	1.00	1.07	0.88	0.99	0.88	1.05	0.97	1.18	0.94	1.08	0.30
7.8	4.4	6.5	2.8	-1.1	-2.1	-3.9	-3.8	-3.9	-3.9	-4.1	-5.7	-2.3	-2.1	-0.6
0.47	0.80	1.23	1.05	1.17	0.92	1.03	0.82	1.03	0.92	1.17	1.05	1.23	0.80	0.47
0.49	0.85	1.31	1.08	1.16	0.91	0.99	0.79	0.98	0.88	1.09	0.99	1.20	0.79	0.47
4.9	6.2	6.3	3.0	-0.5	-1.6	-3.9	-3.8	-5.0	-5.1	-6.2	-5.7	-2.5	-0.9	0.8
0.31	1.11	0.98	1.26	1.01	1.10	0.92	1.03	0.92	1.10	1.01	1.26	0.98	1.11	0.31
0.32	1.16	1.03	1.28	1.00	1.08	0.90	1.00	0.87	1.03	0.94	1.18	0.94	1.10	0.31
4.8	4.9	4.9	2.2	-0.3	-0.9	-2.0	-3.1	-5.5	-5.6	-6.4	-6.3	-4.6	-0.9	1.6
1.02	1.25	1.37	1.22	0.92	1.09	0.92	1.09	0.92	1.22	1.37	1.25	1.02		10
1.07	1.31	1.41	1.25	0.93	1.06	0.89	1.06	0.88	1.15	1.30	1.16	1.06		
4.8	4.8	3.5	2.9	1.2	-2.7	-3.2	-4.8	-4.9	-5.5	-5.1	-4.8	3.8		

```
MAP NO: N2-3- 1          DATE: 5/29/83          POWER:      4%  
CONTROL ROD POSITIONS:   F-Q(T) = 2.479          QPTR:  
D BANK AT 211 STEPS     F-DH(N) = 1.574          NW    1.028 | NE 0.980  
                          F(Z)    = 1.514          -----|-----  
                          F(XY)   = 1.662          SW    1.025 | SE 0.967  
  
BURNUP =                0 MWd/MTU        A.O = 22.96(%)
```

**FIGURE 6.2**  
**NORTH ANNA UNIT 2, CYCLE 3**  
**ASSEMBLYWISE POWER DISTRIBUTION**  
**31% POWER**

R	P	N	N	L	K	J	H	G	F	E	D	C	B	A	
..... PREDICTED . . . . . 0.30 . 0.46 . 0.30 . . . . . PREDICTED . MEASURED . . . . . 0.32 . 0.49 . 0.32 . . . . . MEASURED . PCT DIFFERENCE . . . . . 6.7 . 6.5 . 5.6 . . . . . PCT DIFFERENCE . .....															1
..... 0.57 . 1.04 . 1.10 . 0.76 . 1.10 . 1.04 . 0.37 . . . . . 0.61 . 1.06 . 1.12 . 0.77 . 1.13 . 1.08 . 0.59 . . . . . 6.6 . 2.0 . 1.7 . 1.5 . 2.5 . 4.3 . 4.2 . . . . . .....															2
..... 0.65 . 1.11 . 1.26 . 0.98 . 1.23 . 0.98 . 1.26 . 1.11 . 0.65 . . . . . 0.70 . 1.18 . 1.29 . 0.97 . 1.21 . 0.98 . 1.32 . 1.15 . 0.61 . . . . . 6.5 . 6.5 . 2.0 . -1.2 . -1.5 . -0.7 . 4.2 . 4.2 . -3.2 . . . . . .....															3
..... 0.65 . 1.18 . 1.30 . 1.38 . 1.26 . 1.05 . 1.26 . 1.38 . 1.30 . 1.18 . 0.65 . . . . . 0.69 . 1.22 . 1.34 . 1.38 . 1.24 . 1.03 . 1.22 . 1.35 . 1.27 . 1.15 . 0.64 . . . . . 5.3 . 3.6 . 3.1 . -0.2 . -1.6 . -1.7 . -3.1 . -2.2 . -2.7 . -2.8 . 0.6 . . . . . .....															4
..... 0.57 . 1.10 . 1.30 . 1.00 . 1.21 . 1.00 . 1.16 . 1.00 . 1.21 . 1.00 . 1.30 . 1.10 . 0.57 . . . . . 0.59 . 1.15 . 1.31 . 0.99 . 1.19 . 0.98 . 1.14 . 0.99 . 1.21 . 0.99 . 1.26 . 1.11 . 0.59 . . . . . 4.2 . 4.2 . 0.6 . -1.1 . -1.6 . -2.3 . -2.3 . -1.2 . -0.7 . -1.5 . -2.8 . 0.6 . 4.5 . . . . . .....															5
..... 1.03 . 1.26 . 1.38 . 1.21 . 0.88 . 1.08 . 0.92 . 1.08 . 4.88 . 1.21 . 1.38 . 1.26 . 1.03 . . . . . 1.08 . 1.72 . 1.40 . 1.20 . 0.86 . 1.05 . 0.90 . 1.07 . 0.87 . 1.19 . 1.34 . 1.27 . 1.07 . . . . . 5.0 . 4.9 . 1.8 . -1.3 . -1.9 . -2.2 . -1.8 . -0.7 . -1.1 . -1.4 . -2.7 . 1.1 . 3.2 . . . . . .....															6
..... 0.30 . 1.10 . 0.98 . 1.26 . 1.00 . 1.08 . 0.91 . 1.03 . 0.91 . 1.08 . 1.00 . 1.26 . 0.98 . 1.10 . 0.30 . . . . . 0.32 . 1.14 . 1.02 . 1.27 . 0.99 . 1.06 . 0.89 . 1.01 . 0.90 . 1.07 . 1.00 . 1.22 . 0.98 . 1.09 . 0.31 . . . . . 5.7 . 3.5 . 5.7 . 1.0 . -1.7 . -1.9 . -2.1 . -1.6 . -1.0 . -0.8 . -0.5 . -2.9 . -0.5 . -0.2 . 2.1 . . . . . .....															7
..... 0.46 . 0.76 . 1.22 . 1.05 . 1.17 . 0.92 . 1.02 . 0.82 . 1.02 . 0.92 . 1.17 . 1.05 . 1.22 . 0.76 . 0.46 . . . . . 0.46 . 0.78 . 1.26 . 1.07 . 1.16 . 0.91 . 1.00 . 0.80 . 1.01 . 0.90 . 1.14 . 1.02 . 1.21 . 0.78 . 0.48 . . . . . 1.1 . 3.3 . 3.3 . 1.6 . -0.3 . -0.9 . -2.1 . -1.5 . -1.7 . -1.8 . -2.3 . -2.9 . -0.7 . 2.3 . 5.1 . . . . . .....															8
..... 0.30 . 1.10 . 0.98 . 1.26 . 1.00 . 1.08 . 0.91 . 1.03 . 0.91 . 1.08 . 1.00 . 1.26 . 0.98 . 1.10 . 0.30 . . . . . 0.30 . 1.11 . 0.99 . 1.27 . 1.00 . 1.07 . 0.90 . 1.01 . 0.89 . 1.06 . 0.97 . 1.20 . 0.94 . 1.13 . 0.32 . . . . . 0.7 . 0.9 . 1.0 . 0.4 . -0.2 . -0.7 . -1.5 . -1.4 . -2.1 . -2.2 . -3.5 . -4.4 . -4.3 . 2.6 . 7.6 . . . . . .....															9
..... 1.03 . 1.26 . 1.38 . 1.21 . 0.88 . 1.08 . 0.92 . 1.08 . 0.88 . 1.21 . 1.38 . 1.26 . 1.03 . . . . . 1.04 . 1.27 . 1.39 . 1.22 . 0.88 . 1.05 . 0.90 . 1.05 . 0.85 . 1.17 . 1.31 . 1.18 . 1.11 . . . . . 0.6 . 0.6 . 0.8 . 0.9 . -0.1 . -2.4 . -2.1 . -2.6 . -2.6 . -3.9 . -4.6 . -5.8 . 7.9 . . . . . .....															10
..... 0.57 . 1.10 . 1.30 . 1.00 . 1.21 . 1.00 . 1.16 . 1.00 . 1.21 . 1.00 . 1.30 . 1.10 . 0.57 . . . . . 0.59 . 1.15 . 1.34 . 1.02 . 1.21 . 0.98 . 1.14 . 0.97 . 1.18 . 0.97 . 1.26 . 1.08 . 0.58 . . . . . 4.0 . 4.0 . 3.1 . 1.3 . -0.6 . -2.1 . -2.2 . -3.4 . -3.1 . -3.9 . -2.6 . -2.3 . 1.5 . . . . . .....															11
..... 0.65 . 1.18 . 1.30 . 1.38 . 1.26 . 1.05 . 1.26 . 1.38 . 1.30 . 1.18 . 0.65 . . . . . 0.70 . 1.23 . 1.32 . 1.38 . 1.23 . 1.03 . 1.23 . 1.34 . 1.27 . 1.16 . 0.65 . . . . . 7.4 . 4.5 . 1.3 . -0.2 . -2.2 . -2.3 . -2.6 . -2.9 . -2.9 . -1.6 . -0.5 . . . . . .....															12
..... 0.65 . 1.11 . 1.26 . 0.98 . 1.23 . 0.98 . 1.26 . 1.11 . 0.65 . . . . . 0.71 . 1.20 . 1.31 . 0.97 . 1.22 . 0.98 . 1.26 . 1.07 . 0.65 . . . . . 7.9 . 0.4 . 1.6 . -1.8 . -0.5 . -0.7 . -0.4 . -2.8 . -0.8 . . . . . .....															13
..... 0.57 . 1.04 . 1.10 . 0.76 . 1.10 . 1.04 . 0.37 . . . . . 0.62 . 1.12 . 1.08 . 0.76 . 1.10 . 1.03 . 0.55 . . . . . 0.4 . 0.5 . -2.0 . 0.2 . -0.4 . -0.1 . -2.7 . . . . . .....															14
..... STANDARD . . . . . 0.30 . 0.46 . 0.30 . . . . . AVERAGE . DEVIATION . . . . . 0.33 . 0.47 . 0.31 . . . . . PCT DIFFERENCE . 1.998 . . . . . 8.6 . 2.6 . 2.5 . . . . . * 2.6 . .....															15

**SUMMARY**

MAP NO: N2-3- 2	DATE: 5/29/83	POWER: 31%
CONTROL ROD POSITIONS:	F-Q(T) = 2.279	QPTR:
D BANK AT 177 STEPS	F-DH(N) = 1.570	NW 1.010   NE 0.994
	F(Z) = 1.386	----- -----
	F(XY) = 1.646	SW 1.016   SE 0.980
BURNUP =	0 MW/MTU	A.O = 0.60(%)

# FIGURE 6.3 NORTH ANNA UNIT 2, CYCLE 3 ASSEMBLYWISE POWER DISTRIBUTION 50% POWER

R	P	N	M	L	K	J	H	G	F	E	D	C	B	A	
..... . PREDICTED . . MEASURED . . PCT DIFFERENCE . .....															1
..... . 0.31 . 0.47 . 0.31 . . 0.32 . 0.45 . 0.33 . . 2.3 . -3.8 . 3.7 . .....															
..... . 0.55 . 1.00 . 1.09 . 0.78 . 1.09 . 1.00 . 0.55 . . 0.40 . 1.03 . 1.08 . 0.77 . 1.09 . 1.04 . 0.58 . . 7.9 . 2.4 . -0.6 . -0.9 . 0.1 . 3.5 . 5.2 . .....															2
..... . 0.63 . 1.06 . 1.22 . 0.99 . 1.22 . 0.99 . 1.22 . 1.06 . 0.63 . . 0.66 . 1.10 . 1.23 . 0.99 . 1.21 . 0.99 . 1.22 . 1.09 . 0.67 . . 4.4 . 4.0 . 1.4 . -0.4 . -0.9 . -0.8 . 0.6 . 2.6 . 7.1 . .....															3
..... . 0.63 . 1.12 . 1.25 . 1.34 . 1.26 . 1.08 . 1.26 . 1.34 . 1.25 . 1.12 . 0.63 . . 0.66 . 1.16 . 1.28 . 1.34 . 1.24 . 1.07 . 1.22 . 1.32 . 1.26 . 1.14 . 0.66 . . 7.6 . 3.2 . 2.8 . 0.0 . -1.6 . -1.7 . -2.9 . -1.7 . 0.7 . 1.8 . 4.3 . .....															4
..... . 0.55 . 1.05 . 1.25 . 1.01 . 1.21 . 1.05 . 1.20 . 1.05 . 1.21 . 1.01 . 1.25 . 1.05 . 0.55 . . 0.57 . 1.09 . 1.25 . 1.00 . 1.20 . 1.01 . 1.15 . 1.02 . 1.20 . 1.00 . 1.24 . 1.08 . 0.59 . . 3.6 . 3.6 . 0.0 . -0.6 . -1.5 . -3.7 . -3.7 . -2.3 . -1.3 . -1.1 . -0.8 . 3.0 . 6.8 . .....															5
..... . 1.00 . 1.21 . 1.33 . 1.21 . 0.93 . 1.13 . 0.99 . 1.13 . 0.93 . 1.21 . 1.33 . 1.21 . 1.00 . . 1.04 . 1.26 . 1.35 . 1.20 . 0.90 . 1.08 . 0.95 . 1.10 . 0.91 . 1.19 . 1.30 . 1.22 . 1.04 . . 4.1 . 4.1 . 1.4 . -1.4 . -2.9 . -4.0 . -3.6 . -2.5 . -1.7 . -1.8 . -2.3 . 0.7 . 4.5 . .....															6
..... . 0.31 . 1.08 . 0.99 . 1.26 . 1.05 . 1.13 . 0.99 . 1.10 . 0.99 . 1.13 . 1.05 . 1.26 . 0.99 . 0.31 . . 0.35 . 1.13 . 1.01 . 1.25 . 1.01 . 1.09 . 0.95 . 1.06 . 0.96 . 1.11 . 1.01 . 1.21 . 0.99 . 0.32 . . 10.3 . 4.8 . 2.2 . -0.7 . -3.6 . -3.9 . -4.2 . -3.6 . -2.7 . -2.4 . -3.1 . -4.1 . -0.7 . 1.0 . 2.7 . .....															7
..... . 0.47 . 0.78 . 1.22 . 1.06 . 1.20 . 0.99 . 1.10 . 0.90 . 1.10 . 0.99 . 1.20 . 1.06 . 1.22 . 0.78 . 0.47 . . 0.51 . 0.81 . 1.24 . 1.08 . 1.16 . 0.96 . 1.05 . 0.87 . 1.06 . 0.95 . 1.14 . 1.04 . 1.21 . 0.79 . 0.48 . . 10.1 . 4.5 . 1.6 . -0.5 . -2.9 . -3.4 . -4.5 . -3.5 . -3.7 . -3.7 . -4.6 . -4.0 . -0.7 . 1.1 . 2.7 . .....															8
..... . 0.31 . 1.08 . 0.99 . 1.26 . 1.05 . 1.13 . 0.99 . 1.10 . 0.99 . 1.13 . 1.05 . 1.26 . 0.99 . 0.31 . . 0.35 . 1.12 . 1.00 . 1.24 . 1.02 . 1.09 . 0.94 . 1.05 . 0.95 . 1.08 . 1.00 . 1.22 . 1.00 . 0.32 . . 10.3 . 3.8 . 0.5 . -1.2 . -2.8 . -3.6 . -5.0 . -4.2 . -4.2 . -4.3 . -4.5 . -2.9 . 1.0 . 1.2 . 3.2 . .....															9
..... . 1.00 . 1.21 . 1.33 . 1.21 . 0.93 . 1.13 . 0.99 . 1.13 . 0.93 . 1.21 . 1.33 . 1.21 . 1.00 . . 1.02 . 1.23 . 1.34 . 1.21 . 0.91 . 1.07 . 0.95 . 1.07 . 0.89 . 1.17 . 1.33 . 1.25 . 1.03 . . 1.6 . 1.6 . 0.4 . -0.3 . -2.0 . -5.1 . -4.2 . -4.9 . -4.0 . -3.8 . -0.5 . 2.9 . 3.0 . .....															10
..... . 0.55 . 1.05 . 1.25 . 1.01 . 1.21 . 1.05 . 1.20 . 1.05 . 1.21 . 1.01 . 1.25 . 1.05 . 0.55 . . 0.56 . 1.11 . 1.29 . 1.02 . 1.19 . 1.00 . 1.15 . 0.99 . 1.17 . 0.99 . 1.27 . 1.08 . 0.57 . . 5.1 . 5.1 . 3.9 . 1.5 . -2.0 . -3.9 . -3.9 . -5.5 . -3.1 . -1.7 . 1.6 . 2.9 . 3.9 . .....															11
..... . 0.63 . 1.12 . 1.25 . 1.34 . 1.26 . 1.08 . 1.26 . 1.34 . 1.25 . 1.12 . 0.63 . . 0.69 . 1.18 . 1.27 . 1.33 . 1.23 . 1.06 . 1.22 . 1.31 . 1.23 . 1.14 . 0.65 . . 8.7 . 5.2 . 1.5 . -0.7 . -2.4 . -2.4 . -2.8 . -2.4 . -1.5 . 1.7 . 2.9 . .....															12
..... . 0.63 . 1.06 . 1.22 . 0.99 . 1.22 . 0.99 . 1.22 . 1.06 . 0.63 . . 0.69 . 1.16 . 1.26 . 0.97 . 1.21 . 0.99 . 1.22 . 1.05 . 0.65 . . 9.1 . 9.6 . 3.4 . -2.4 . -1.2 . -0.5 . 0.1 . -0.2 . 2.7 . .....															13
..... . 0.55 . 1.00 . 1.09 . 0.78 . 1.09 . 1.00 . 0.55 . . 0.61 . 1.11 . 1.14 . 0.81 . 1.08 . 1.00 . 0.55 . . 9.6 . 11.1 . 5.4 . 3.8 . -0.3 . 0.2 . -1.1 . .....															14
..... . STANDARD . . DEVIATION . . =2.373 . .....															15
..... . 0.31 . 0.47 . 0.31 . . 0.35 . 0.50 . 0.32 . . 12.4 . 7.2 . 1.6 . .....															
..... . AVERAGE . . PCT DIFFERENCE . . = 3.1 . .....															

## SUMMARY

MAP NO: N2-3- 3	DATE: 6/ 6/83	POWER: 50%
CONTROL ROD POSITIONS:	F-Q(T) = 2.139	QPTR:
D BANK AT 179 STEPS	F-DH(N) = 1.545	NW 1.008   NE 0.994
	F(Z) = 1.320	----- -----
	F(XY) = 1.620	SW 1.015   SE 0.983
BURNUP =	85 MW/MTU	A.O = -1.82(%)

FIGURE 6.4  
NORTH ANNA UNIT 2, CYCLE 3  
ASSEMBLYWISE POWER DISTRIBUTION  
79% POWER

R	P	N	M	L	K	J	H	G	F	E	D	C	B	A	
..... . PREDICTED . . MEASURED . . PCT DIFFERENCE . .....															1
..... . 0.34 . 0.51 . 0.34 . . 0.34 . 0.52 . 0.35 . . 2.5 . 2.3 . 3.0 . .....															
..... . 0.56 . 0.99 . 1.10 . 0.83 . 1.10 . 0.99 . 0.56 . . 0.60 . 1.01 . 1.09 . 0.82 . 1.11 . 1.04 . 0.59 . . 7.3 . 1.5 . -0.2 . -0.6 . 1.0 . 5.0 . 3.9 . .....															2
..... . 0.63 . 1.04 . 1.18 . 1.00 . 1.23 . 1.00 . 1.18 . 1.04 . 0.63 . . 0.66 . 1.09 . 1.20 . 0.98 . 1.20 . 0.99 . 1.21 . 1.08 . 0.66 . . 4.9 . 4.7 . 1.8 . -2.0 . -2.3 . -1.0 . 2.1 . -3.8 . 6.9 . .....															3
..... . 0.63 . 1.10 . 1.21 . 1.30 . 1.24 . 1.09 . 1.24 . 1.30 . 1.21 . 1.10 . 0.63 . . 0.67 . 1.14 . 1.25 . 1.30 . 1.20 . 1.06 . 1.20 . 1.30 . 1.23 . 1.13 . 0.66 . . 5.5 . 3.5 . 3.2 . 0.2 . -2.5 . -2.6 . -2.9 . -0.2 . 1.9 . 2.9 . 4.5 . .....															4
..... . 0.54 . 1.04 . 1.21 . 0.99 . 1.19 . 1.05 . 1.20 . 1.05 . 1.19 . 0.99 . 1.21 . 1.04 . 0.54 . . 0.58 . 1.08 . 1.22 . 1.00 . 1.19 . 1.01 . 1.16 . 1.03 . 1.19 . 1.00 . 1.22 . 1.07 . 0.59 . . 3.8 . 3.7 . 1.3 . 0.4 . -0.6 . -3.7 . -3.7 . -2.0 . -0.0 . 0.3 . 0.8 . 3.3 . 5.5 . .....															5
..... . 0.99 . 1.18 . 1.30 . 1.19 . 0.96 . 1.14 . 1.02 . 1.14 . 0.94 . 1.19 . 1.30 . 1.18 . 0.99 . . 1.03 . 1.23 . 1.32 . 1.19 . 0.93 . 1.09 . 0.97 . 1.11 . 0.94 . 1.19 . 1.29 . 1.20 . 1.03 . . 3.9 . 3.8 . 1.7 . -0.5 . -2.6 . -4.4 . -4.1 . -2.5 . -1.1 . -0.3 . -0.6 . 1.3 . 3.8 . .....															6
..... . 0.34 . 1.10 . 1.00 . 1.24 . 1.05 . 1.15 . 1.02 . 1.14 . 1.02 . 1.13 . 1.05 . 1.24 . 1.00 . 0.34 . . 0.35 . 1.12 . 1.01 . 1.23 . 1.02 . 1.10 . 0.97 . 1.09 . 1.00 . 1.12 . 1.03 . 1.19 . 0.99 . 0.34 . . 4.9 . 2.5 . 1.4 . -0.7 . -2.7 . -3.7 . -4.8 . -4.3 . -2.9 . -2.4 . -2.9 . -3.4 . -0.7 . 0.4 . 2.1 . .....															7
..... . 0.51 . 0.82 . 1.23 . 1.09 . 1.20 . 1.01 . 1.13 . 0.95 . 1.13 . 1.01 . 1.20 . 1.09 . 1.23 . 0.82 . 0.51 . . 0.53 . 0.84 . 1.24 . 1.08 . 1.17 . 0.98 . 1.07 . 0.91 . 1.09 . 0.97 . 1.16 . 1.05 . 1.22 . 0.83 . 0.52 . . 4.8 . 2.2 . 1.0 . -0.8 . -2.7 . -3.5 . -5.2 . -4.2 . -3.9 . -3.9 . -3.9 . -3.4 . -0.8 . 1.2 . 2.9 . .....															8
..... . 0.34 . 1.10 . 1.00 . 1.24 . 1.05 . 1.15 . 1.02 . 1.14 . 1.02 . 1.15 . 1.05 . 1.24 . 1.00 . 0.34 . . 0.35 . 1.12 . 1.01 . 1.22 . 1.03 . 1.11 . 0.99 . 1.09 . 0.98 . 1.10 . 1.01 . 1.20 . 1.00 . 0.35 . . 4.9 . 2.2 . 0.7 . -0.9 . -2.5 . -2.9 . -3.4 . -3.7 . -4.2 . -4.2 . -3.8 . -3.2 . -0.2 . 1.8 . 3.5 . .....															9
..... . 0.99 . 1.18 . 1.30 . 1.19 . 0.96 . 1.14 . 1.02 . 1.14 . 0.96 . 1.19 . 1.30 . 1.18 . 0.99 . . 1.02 . 1.22 . 1.31 . 1.19 . 0.94 . 1.09 . 0.97 . 1.09 . 0.92 . 1.16 . 1.27 . 1.19 . 1.03 . . 3.0 . 2.9 . 1.0 . 0.0 . -1.3 . -4.8 . -4.6 . -4.6 . -3.3 . -2.8 . -1.9 . 0.2 . 4.0 . .....															10
..... . 0.54 . 1.04 . 1.21 . 0.99 . 1.19 . 1.05 . 1.20 . 1.05 . 1.19 . 0.99 . 1.21 . 1.04 . 0.54 . . 0.59 . 1.09 . 1.25 . 1.01 . 1.19 . 1.02 . 1.16 . 1.00 . 1.17 . 0.98 . 1.21 . 1.05 . 0.58 . . 5.0 . 5.0 . 3.7 . 1.3 . -0.7 . -3.3 . -3.5 . -5.0 . -2.2 . -1.5 . 0.5 . 1.6 . 4.2 . .....															11
..... . 0.63 . 1.10 . 1.21 . 1.30 . 1.24 . 1.09 . 1.24 . 1.30 . 1.21 . 1.10 . 0.63 . . 0.68 . 1.15 . 1.22 . 1.31 . 1.20 . 1.05 . 1.20 . 1.29 . 1.20 . 1.12 . 0.65 . . 7.1 . 4.3 . 1.2 . 0.5 . -3.2 . -3.4 . -3.2 . -1.1 . -0.6 . 1.7 . 3.1 . .....															12
..... . 0.63 . 1.04 . 1.18 . 1.00 . 1.23 . 1.00 . 1.18 . 1.04 . 0.63 . . 0.68 . 1.11 . 1.22 . 0.98 . 1.22 . 1.00 . 1.20 . 1.05 . 0.65 . . 7.0 . 7.0 . 3.4 . -1.9 . -1.0 . -0.4 . 1.2 . 0.9 . 2.9 . .....															13
..... . 0.54 . 0.99 . 1.10 . 0.83 . 1.10 . 0.99 . 0.54 . . 0.60 . 1.08 . 1.13 . 0.84 . 1.09 . 1.00 . 0.54 . . 7.0 . 8.3 . 3.1 . 2.2 . -0.2 . 1.2 . 0.9 . .....															14
..... . 0.34 . 0.51 . 0.34 . . 0.37 . 0.54 . 0.34 . . 9.5 . 5.6 . 1.6 . .....															
..... . STANDARD . . DEVIATION . . =1.862 . .....															
..... . AVERAGE . . PCT DIFFERENCE . . = 2.8 . .....															15

SUMMARY

MAP NO: N2-3- 6	DATE: 6/ 7/83	POWER: 79%
CONTROL ROD POSITIONS:	F-Q(T) = 1.980	QPTR:
O BANK AT 192 STEPS	F-DH(N) = 1.480	NW 1.004   NE 0.998
	F(Z) = 1.270	----- -----
	F(XY) = 1.555	SW 1.011   SE 0.987
	BURNUP = 112 MSB/MTU	A.O = -0.30(%)



	P	N	M	K	J	S	F	T	D	C	B	A
PREDICTED					0.35	0.52	0.35					
MEASURED					0.36	0.54	0.35					
PCT DIFFERENCE					3.3	3.3	1.7					
					0.55	0.99	1.11	0.86	1.11	0.99	0.55	
					0.57	1.00	1.11	0.86	1.10	0.99	0.57	
					2.6	1.2	-0.1	-0.1	-0.5	0.0	3.5	
					0.63	1.02	1.17	1.00	1.24	1.00	1.17	1.02
					0.64	1.04	1.18	0.98	1.21	0.98	1.16	1.04
					2.0	1.9	1.2	-1.9	-2.2	-2.4	-1.1	1.0
					0.63	1.08	1.19	1.29	1.23	1.09	1.23	1.29
					0.65	1.10	1.20	1.30	1.20	1.06	1.19	1.27
					3.5	2.1	1.3	0.9	-2.0	-3.2	-1.2	1.6
					0.63	1.08	1.19	1.29	1.23	1.09	1.23	1.29
					0.65	1.10	1.20	1.30	1.20	1.06	1.19	1.27
					3.5	2.1	1.3	0.9	-2.0	-3.2	-1.2	1.6
					0.55	1.02	1.19	0.99	1.19	1.05	1.20	1.05
					0.58	1.07	1.20	0.99	1.19	1.02	1.16	1.04
					4.3	4.3	0.9	-0.3	-0.3	-3.4	-3.4	-1.8
					0.99	1.17	1.28	1.19	0.99	1.16	1.02	1.16
					1.03	1.22	1.29	1.18	0.97	1.10	0.98	1.13
					4.3	4.3	0.6	-1.2	-2.5	-4.8	-4.6	-2.3
					0.34	1.11	1.00	1.23	1.06	1.16	1.04	1.15
					0.36	1.12	0.98	1.20	1.03	1.12	0.99	1.09
					4.6	1.2	-2.1	-2.6	-2.8	-3.7	-4.8	-4.6
					0.52	0.86	1.24	1.09	1.20	1.02	1.14	1.02
					0.55	0.87	1.21	1.06	1.18	0.99	1.08	0.91
					4.6	1.2	-2.1	-2.1	-2.1	-3.1	-5.3	-4.3
					0.34	1.11	1.00	1.23	1.06	1.16	1.04	1.15
					0.36	1.12	1.00	1.21	1.04	1.13	1.00	1.10
					4.6	1.5	-0.1	-1.1	-1.9	-2.4	-3.3	-3.6
					0.99	1.17	1.28	1.19	0.99	1.16	1.02	1.16
					1.01	1.19	1.29	1.19	0.98	1.12	0.99	1.11
					1.9	1.9	0.2	-0.7	-1.6	-3.3	-3.6	-3.5
					0.55	1.02	1.19	0.99	1.19	1.05	1.20	1.05
					0.58	1.07	1.22	0.98	1.18	1.03	1.18	1.02
					4.4	4.4	2.7	-0.6	-1.4	-1.9	-1.9	-3.4
					0.63	1.08	1.19	1.29	1.23	1.09	1.23	1.29
			</									

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	P	H	M	L	K	J	N	G	F	E	O	C	B	A
PREDICTED						0.35	0.52	0.35						
MEASURED						0.36	0.54	0.36						
PCT DIFFERENCE						3.5	3.5	3.9						
	0.55	0.99	1.11	0.86	1.11	0.99	0.55							
	0.59	0.99	1.11	0.86	1.12	1.03	0.59							
	3.0	0.0	-0.1	-0.1	1.1	4.2	5.6							
	0.63	1.02	1.17	1.00	1.24	1.00	1.17	1.02	0.63					
	0.65	1.06	1.18	0.98	1.21	0.99	1.10	1.06	0.67					
	3.0	3.5	1.0	-2.0	-2.3	-1.3	1.2	3.2	7.1					
	0.63	1.00	1.19	1.29	1.23	1.09	1.23	1.29	1.19	1.00	0.63			
	0.66	1.11	1.22	1.29	1.20	1.06	1.19	1.27	1.21	1.11	0.66			
	4.7	2.9	2.5	0.1	-2.3	-2.4	-3.1	-1.2	1.5	2.9	4.8			
	0.55	1.02	1.19	0.99	1.19	1.05	1.20	1.05	1.19	0.99	1.19	1.02	0.55	
	0.57	1.06	1.20	0.99	1.19	1.02	1.17	1.03	1.18	0.99	1.20	1.06	0.58	
	3.6	3.6	1.0	0.1	-0.6	-3.0	-3.0	-2.0	-0.8	-0.1	0.8	3.7	5.7	
	0.99	1.17	1.28	1.19	0.99	1.16	1.02	1.16	0.99	1.19	1.28	1.17	0.99	
	1.03	1.22	1.30	1.19	0.97	1.11	0.99	1.13	0.98	1.10	1.27	1.18	1.02	
	4.1	4.1	1.7	-0.7	-2.1	-3.6	-3.5	-2.2	-1.1	-0.8	-1.1	1.1	3.7	
	0.34	1.11	1.00	1.23	1.06	1.16	1.04	1.16	1.06	1.23	1.00	1.11	0.34	
	0.36	1.13	1.01	1.22	1.02	1.12	0.99	1.10	1.01	1.13	1.03	0.99	1.11	0.35
	4.4	2.2	1.3	-0.9	-3.0	-3.6	-4.1	-3.8	-2.7	-2.4	-2.9	-3.9	-0.8	0.6
	0.52	0.86	1.24	1.09	1.20	1.02	1.14	0.96	1.16	1.02	1.20	1.09	1.24	0.86
	0.55	0.88	1.25	1.08	1.17	0.99	1.09	0.92	1.10	0.99	1.15	1.05	1.23	0.80
	4.4	2.1	0.7	-0.7	-2.4	-3.1	-4.7	-3.6	-3.5	-3.6	-4.2	-3.9	-0.9	2.1
	0.34	1.11	1.00	1.23	1.06	1.16	1.04	1.16	1.06	1.23	1.00	1.11	0.34	
	0.36	1.12	0.99	1.21	1.03	1.13	1.00	1.11	1.00	1.12	1.02	1.19	1.00	1.13
	4.4	1.1	-0.6	-1.4	-2.1	-2.3	-3.1	-3.1	-3.6	-3.7	-3.5	-2.8	-0.2	2.4
	0.99	1.17	1.28	1.19	0.99	1.16	1.02	1.16	0.99	1.19	1.28	1.17	0.99	
	3.00	1.18	1.29	1.20	0.99	1.11	0.99	1.11	0.97	1.16	1.26	1.17	1.03	
	1.3	1.3	0.9	0.6	-0.7	-3.8	-3.6	-3.7	-2.7	-2.4	-1.4	-0.2	4.4	
	0.55	1.02	1.19	0.99	1.19	1.05	1.20	1.05	1.19	0.99	1.19	1.02	0.55	
	0.57	1.06	1.22	1.01	1.19	1.03	1.17	1.01	1.17	0.98	1.20	1.04	0.58	
	3.7	3.7	3.0	1.6	-0.6	-2.7	-2.9	-4.0	-1.9	-1.3	0.9	1.5	3.9	
	0.63	1.00	1.19	1.29	1.23	1.09	1.23	1.29	1.1					

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FIGURE 6.7  
NORTH ANNA UNIT 2, CYCLE 3  
ASSEMBLYWISE POWER DISTRIBUTION  
100% POWER

R	P	M	N	L	K	J	H	G	F	E	D	C	B	A	
<div> <div>PREDICTED</div> <div>MEASURED</div> <div>PCT DIFFERENCE</div> </div>															
<div> <div>0.34 0.52 0.34</div> <div>0.36 0.54 0.36</div> <div>3.7 3.4 3.4</div> </div>															1
<div> <div>0.56 0.99 1.10 0.85 1.10 0.99 0.56</div> <div>0.59 1.00 1.10 0.85 1.11 1.02 0.58</div> <div>6.3 0.9 0.0 -0.1 0.9 3.4 5.1</div> </div>															2
<div> <div>0.63 1.03 1.17 1.00 1.24 1.00 1.17 1.03 0.63</div> <div>0.65 1.07 1.18 0.98 1.21 0.99 1.18 1.06 0.67</div> <div>4.0 3.7 1.0 -2.0 -2.3 -1.5 0.7 2.9 7.0</div> </div>															3
<div> <div>0.63 1.08 1.19 1.29 1.23 1.09 1.23 1.29 1.19 1.08 0.63</div> <div>0.64 1.12 1.22 1.29 1.20 1.06 1.19 1.25 1.21 1.11 0.66</div> <div>5.0 3.1 2.7 0.2 -2.1 -2.2 -3.1 -1.0 1.4 2.7 4.5</div> </div>															4
<div> <div>0.56 1.03 1.19 0.99 1.19 1.05 1.20 1.05 1.19 0.99 1.19 1.03 0.56</div> <div>0.58 1.06 1.20 0.99 1.19 1.02 1.17 1.03 1.19 0.99 1.19 1.06 0.59</div> <div>3.7 3.7 1.0 0.1 -0.5 -2.9 -2.9 -2.1 -0.6 -0.2 0.4 3.3 5.9</div> </div>															5
<div> <div>0.99 1.17 1.29 1.19 0.98 1.15 1.02 1.15 0.98 1.19 1.29 1.17 0.99</div> <div>1.02 1.21 1.30 1.19 0.96 1.11 0.99 1.13 0.97 1.18 1.27 1.18 1.02</div> <div>3.5 3.5 1.5 -0.5 -2.0 -3.6 -3.3 -2.1 -0.9 -0.8 -1.1 0.8 3.3</div> </div>															6
<div> <div>0.34 1.10 1.00 1.23 1.06 1.16 1.04 1.15 1.04 1.16 1.06 1.23 1.00 1.10 0.34</div> <div>0.36 1.12 1.00 1.21 1.02 1.11 0.99 1.11 1.01 1.13 1.03 1.19 0.99 1.10 0.35</div> <div>5.1 1.9 0.5 -1.3 -3.1 -3.6 -4.3 -3.6 -2.5 -2.0 -2.6 -3.5 -1.1 0.1 1.4</div> </div>															7
<div> <div>0.52 0.85 1.24 1.09 1.20 1.02 1.14 0.96 1.14 1.02 1.20 1.09 1.24 0.85 0.52</div> <div>0.53 0.86 1.24 1.07 1.17 0.99 1.09 0.92 1.11 0.99 1.16 1.05 1.22 0.87 0.55</div> <div>5.0 1.8 0.1 -1.4 -3.0 -3.5 -4.6 -3.4 -3.3 -3.4 -3.9 -3.4 -1.1 2.7 4.9</div> </div>															8
<div> <div>0.34 1.10 1.00 1.23 1.06 1.16 1.04 1.15 1.04 1.16 1.06 1.23 1.00 1.10 0.34</div> <div>0.36 1.12 0.99 1.21 1.03 1.12 1.00 1.11 1.00 1.11 1.02 1.20 1.00 1.14 0.36</div> <div>5.1 1.4 -0.4 -1.7 -2.9 -3.1 -3.4 -3.3 -3.6 -3.7 -3.4 -2.6 -0.3 3.4 5.8</div> </div>															9
<div> <div>0.99 1.17 1.29 1.19 0.98 1.15 1.02 1.15 0.98 1.19 1.29 1.17 0.99</div> <div>1.00 1.19 1.29 1.19 0.97 1.10 0.98 1.10 0.96 1.17 1.27 1.17 1.05</div> <div>1.7 1.7 0.5 -0.1 -1.3 -4.3 -4.0 -4.2 -2.6 -2.1 -1.3 -0.1 6.5</div> </div>															10
<div> <div>0.56 1.03 1.19 0.99 1.19 1.05 1.20 1.05 1.19 0.99 1.19 1.03 0.56</div> <div>0.58 1.07 1.23 1.01 1.18 1.02 1.16 1.00 1.20 0.99 1.20 1.04 0.58</div> <div>4.3 4.3 3.3 1.6 -0.9 -3.2 -3.4 -5.3 0.2 -0.4 1.1 1.7 4.1</div> </div>															11
<div> <div>0.63 1.08 1.19 1.29 1.23 1.09 1.23 1.29 1.19 1.08 0.63</div> <div>0.67 1.13 1.21 1.29 1.19 1.05 1.19 1.29 1.19 1.11 0.65</div> <div>6.9 4.3 1.6 0.2 -3.3 -3.4 -3.3 -0.2 0.1 2.4 3.7</div> </div>															12
<div> <div>0.63 1.03 1.17 1.00 1.24 1.00 1.17 1.03 0.63</div> <div>0.67 1.11 1.21 0.98 1.21 0.99 1.17 1.13 0.65</div> <div>7.3 7.8 3.4 -8.5 -1.8 -1.5 -0.3 0.1 3.4</div> </div>															13
<div> <div>0.56 0.99 1.10 0.85 1.10 0.99 0.56</div> <div>0.60 1.07 1.14 0.87 1.09 0.99 0.55</div> <div>7.8 8.8 3.1 1.8 -1.4 -0.3 -0.5</div> </div>															14
<div> <div>STANDARD</div> <div>DEVIATION</div> <div>=1.922</div> </div>															15
<div> <div>0.34 0.52 0.34</div> <div>0.36 0.55 0.34</div> <div>9.8 5.1 0.1</div> </div>															
<div> <div>AVERAGE</div> <div>PCT DIFFERENCE</div> <div>= 2.7</div> </div>															

SUMMARY

MAP NO: N2-3- 9	DATE: 6/ 8/83	POWER: 100%
CONTROL ROD POSITIONS:	F-Q(T) = 1.976	QPTR:
D BANK AT 210 STEPS	F-DH(N) = 1.476	NW 1.003   NE 0.996
	F(Z) = 1.275	----- -----
	F(XY) = 1.555	SW 1.010   SE 0.991
BURNUP = 185 MWD/MTU	A.O = -7.36(%)	

FIGURE 6.8  
NORTH ANNA UNIT 2, CYCLE 3  
ASSEMBLYWISE POWER DISTRIBUTION  
HFP, EQUILIBRIUM XENON

R	P	N	M	L	K	J	H	G	F	E	D	C	B	A					
PREDICTED						0.34	0.52	0.34	PREDICTED										
MEASURED						0.36	0.54	0.36	MEASURED										
PCT DIFFERENCE						3.3	3.3	3.3	PCT DIFFERENCE						1				
.....																			
0.55						0.99	1.11	0.86	1.11	0.99	0.55								
0.59						1.00	1.11	0.86	1.11	1.02	0.58								
0.6						1.9	0.2	0.2	0.8	3.4	5.0								
.....																			
0.63						1.02	1.17	1.00	1.24	1.00	1.17	1.02	0.63						
0.65						1.06	1.19	0.98	1.21	0.98	1.18	1.05	0.67						
4.2						3.9	1.5	-1.8	-2.1	-1.7	0.6	-2.7	6.7	3					
.....																			
0.63						1.08	1.19	1.29	1.23	1.09	1.23	1.29	1.19	1.08	0.63				
0.66						1.11	1.22	1.30	1.20	1.06	1.19	1.27	1.20	1.11	0.65				
4.9						3.0	2.6	1.2	-2.0	-2.2	-3.6	-1.4	1.2	2.5	4.5				
.....																			
0.55						1.02	1.19	0.99	1.20	1.05	1.20	0.99	1.19	1.02	0.55				
0.57						1.06	1.20	0.99	1.20	1.02	1.16	1.03	1.19	0.99	1.19	1.06	0.59		
3.4						3.4	0.9	0.1	0.0	-3.7	-3.7	-2.3	-0.7	-0.3	0.3	3.4	6.0		
.....																			
0.98						1.17	1.28	1.20	0.99	1.16	1.03	1.16	0.99	1.20	1.28	1.17	0.98		
1.02						1.21	1.30	1.19	0.97	1.11	0.99	1.13	0.99	1.19	1.27	1.18	1.02		
3.3						3.3	1.3	-0.7	-1.9	-4.3	-3.8	-2.1	-0.8	-0.8	-1.2	0.9	3.5		
.....																			
0.34						1.10	1.00	1.23	1.06	1.16	1.04	1.15	1.04	1.16	1.06	1.23	1.00	1.10	0.34
0.36						1.12	1.00	1.21	1.03	1.12	0.99	1.10	1.01	1.14	1.03	1.18	0.99	1.11	0.35
5.0						1.9	0.5	-1.1	-2.6	-3.3	-4.3	-3.8	-2.4	-1.8	-2.4	-3.6	-0.9	0.4	1.8
.....																			
0.52						0.86	1.24	1.09	1.20	1.02	1.14	0.94	1.14	1.02	1.20	1.09	1.24	0.86	0.52
0.55						0.88	1.24	1.07	1.17	0.99	1.09	0.92	1.11	0.99	1.16	1.05	1.23	0.88	0.54
3.0						1.8	3.1	-1.1	-2.4	-3.1	-4.6	-3.6	-3.4	-3.4	-3.8	-3.5	-0.9	1.9	3.6
.....																			
0.34						1.10	1.00	1.23	1.06	1.16	1.04	1.15	1.04	1.16	1.06	1.23	1.00	1.10	0.34
0.36						1.12	0.99	1.21	1.03	1.13	1.01	1.11	1.00	1.12	1.02	1.19	1.00	1.13	0.36
5.0						1.4	-0.4	-1.4	-2.2	-2.4	-2.8	-3.1	-3.6	-3.6	-3.3	-2.7	-0.2	2.2	4.1
.....																			
0.98						1.17	1.28	1.20	0.99	1.16	1.03	1.16	0.99	1.20	1.28	1.17	0.98		
1.00						1.19	1.29	1.20	0.98	1.11	0.98	1.11	0.96	1.16	1.26	1.16	1.03		
1.8						1.8	0.7	0.1	-0.9	-4.0	-4.0	-4.2	-2.9	-2.6	-1.4	-0.3	4.2		
.....																			
0.55						1.02	1.19	0.99	1.20	1.05	1.20	1.05	1.20	0.99	1.19	1.02	0.55		
0.58						1.06	1.22	1.00	1.19	1.02	1.16	1.01	1.17	0.98	1.20	1.04	0.57		
4.1						4.1	3.2	1.4	-0.9	-3.3	-3.4	-4.7	-2.0	-1.4	1.0	1.6	3.9		
.....																			
0.63						1.08	1.19	1.29	1.23	1.09	1.23	1.29	1.19	1.08	0.63				
0.67						1.12	1.21	1.29	1.18	1.05	1.19	1.27	1.18	1.10	0.65				
4.5						4.1	1.4	-0.0	-3.5	-3.7	-3.2	-1.1	-0.6	8.2	3.6				
.....																			
0.63						1.02	1.17	1.00	1.24	1.00	1.17	1.02	0.63						
0.67						1.10	1.21	0.97	1.21	0.99	1.17	1.03	0.65						
7.2						7.9	3.3	-2.7	-2.0	-1.2	0.1	0.5	3.3						
.....																			
0.55						0.99	1.11	0.86	1.11	0.99	0.55								
0.60						1.07	1.14	0.83	1.09	0.99	0.56								
7.9						8.6	3.1	1.7	-1.2	0.1	0.4								
.....																			
STANDARD						0.34						0.52	0.34	AVERAGE					
DEVIATION						0.38						0.55	0.34	PCT DIFFERENCE					
=1.831						9.7						4.9	-0.2	= 2.6					
.....																			

SUMMARY

MAP NO: N2-3-10      DATE: 6/10/83      POWER: 100%  
CONTROL ROD POSITIONS: F-Q(T) = 1.906      QPTR:  
D BANK AT 228 STEPS      F-OH(N) = 1.473      NW 1.007 | NE 0.999  
F(Z) = 1.229      SW 1.009 | SE 0.985  
F(XY) = 1.558  
BURNUP = 215 MWD/MTU      A.O = -2.10(%)

## Section 7

### REFERENCES

1. A. Abbasi, R. A. Hall, "North Anna Unit 2, Cycle 3, Design Report," NFE Technical Report No. 288, Vepco, April, 1983.
2. North Anna Unit 2 Technical Specifications, Sections 3.1.3.4, 3/4.2.
3. T. K. Ross, W. C. Beck, "Control Rod Reactivity Worth Determination By The Rod Swap Technique," VEP-FRD-36A, December 1980.
4. T. J. Kunsitis, "RXFLOW, A Computer Program to Calculate Reactor Flow and Thermal Output," NFO-CCR-8, Vepco, March, 1983.
5. "Technical Manual for Westinghouse Solid State Reactivity Computer," Westinghouse Electric Corporation.
6. W. Leggett and L. Eisenhart, "The INCORE Code," WCAP-7149, December, 1967

## APPENDIX

### STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEETS

NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description: Reactivity Computer Checkout Proc No /Section: 2-PT-94.2 Sequence Step No: 3			
II Test Conditions (Design)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB: 228 CC: 228 CD: *	RCS Temperature (°F): 547 Power Level (% F.P.): 0 Other (specify): Below Nuclear Heating *At the just crit. position		
III Test Conditions (Actual)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB: 228 CC: 228 CD: 101	RCS Temperature (°F): 548.1 Power Level (% F.P.): 0 Other (Specify): Below Nuclear Heating		
IV    Test Results	Date/Time Test Performed: 5/27/83 1338			
	Measured Parameter (Description)	$\rho_c$ = Meas. Reactivity using $\rho$ -computer $\rho_t$ = Inferred React from react period		
	Measured Value	$\rho_c = +27 \mu\text{m} \quad -18.5 \quad +44 \quad -27$ $\rho_t = +26.3 \mu\text{m} \quad -18.3 \quad +44 \quad -27.3$ $\%D = 2.7\% \quad 1.1\% \quad 0.0\% \quad -1.1\%$		
		Design Value (Actual Conditions)	$\%D = [(\rho_c - \rho_t) / \rho_t] \times 100\% \leq 4.0\%$	
		Design Value (Design Conditions)	$\%D = [(\rho_c - \rho_t) / \rho_t] \times 100\% \leq 4.0\%$	
	Reference	WCAP 7905, Rev. 1, Table 3.6		
V Acceptance Criteria	FSAR/Tech Spec	Not Applicable		
	Reference	Not Applicable		
VI Comments	Design Tolerance is met : <u>X</u> YES <u>  </u> NO Acceptance Criteria as met : <u>X</u> YES <u>  </u> NO  Allowable Range = $\pm 27 \mu\text{m}$			

Completed By: QST Thomas  
Test Engineer

Evaluated By: C.A. Ford

Recommended for  
Approval By: C.J. Snow  
NFO Engineer

NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description: Critical Boron Concentration - ARO Proc No /Section: 2-PT-94.3 Sequence Step No: 4	
II Test Conditions (Design)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB: 228 CC: 228 CD: 228	RCS Temperature (°F): 547 Power Level (% F.P.): 0 Other (specify): Below Nuclear Heating
III Test Conditions (Actual)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB: 228 CC: 228 CD: 228	RCS Temperature (°F): 542.1 Power Level (% F.P.): 0 Other (Specify): Below Nuclear Heating
	Date/Time Test Performed: 5/28/83 0640	
IV	Meas Parameter (Description)	$(C_B)^M_{ARO}$ ; Critical Boron Conc - ARO
Test Results	Measured Value	$(C_B)^M_{ARO} = 1575 \text{ ppm}$
	Design Value (Actual Cond)	$C_B = 1546 \pm 50 \text{ ppm}$
	Design Value (Design Cond)	$C_B = 1546 \pm 50 \text{ ppm}$
	Reference	VEP-PSE-NFE-268
V	FSAR/Tech Spec	$\alpha_{C_B} \times C_B \leq 24,000 \text{ pcm}$
Acceptance Criteria	Reference	UFSAR Section 15.2.4
VI	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	
Comments	$\alpha_{C_B} = -7.94 \text{ pcm/ppm}$ for preliminary analysis $\alpha_{C_B} = -7.85 \text{ pcm/ppm}$ for final analysis.	

Completed By: B. Thomas  
Test Engineer

Evaluated By: B.D. Mann

Recommended for  
Approval By: C. J. Snow  
NFO Engineer



NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description: Isothermal Temperature Coefficient - ARO Proc No /Section: 2-PT-94.4 Sequence Step No: 5	
II Test Conditions (Design)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB: 228 CC: 228 CD: 228	RCS Temperature (°F): 547 Power Level (% F.P.): 0 Other (specify): Below Nuclear Heating
III Test Conditions (Actual)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB: 228 CC: 228 CD: 212	RCS Temperature (°F): 543.6 Power Level (% F.P.): 0 Other (Specify): Below Nuclear Heating
	Date/Time Test Performed: 5/28/83 0746	
IV	Meas Parameter (Description)	$(\alpha_T^{ISO})_{ARO}$ Isothermal Temp Coeff - ARO
Test Results	Measured Value	$(\alpha_T^{ISO})_{ARO} = -3.65 \text{ pcm/°F } (C_B = 1575 \text{ ppm})$
	Design Value (Actual Cond)	$(\alpha_T^{ISO})_{ARO} = -3.70 \pm 3.0 \text{ pcm/°F } (C_B = 1575 \text{ ppm})$
	Design Value (Design Cond)	$(\alpha_T^{ISO})_{ARO} = -4.04 \pm 3.0 \text{ pcm/°F } (C_B = 1549 \text{ ppm})$
	Reference	VEP-PSE-NFE-288
V Acceptance Criteria	FSAR/Tech Spec	$\alpha_T^{ISO} \leq -2.11 \text{ pcm/°F } \quad \alpha_T^{Dop} = -2.11 \text{ pcm/°F}$
	Reference	TS 3.1.1.4, VEP-PSE-NFE-288
VI	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	
Comments		

Completed By: RS Thomas  
Test Engineer

Evaluated By: B. D. Mann

Recommended for  
Approval By: C. J. Snow  
NFO Engineer



NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description: Cntl Bank B Worth Meas., Rod Swap Ref. Bank Proc No /Section: 2-PT-94.5 Sequence Step No: 7	
II Test Conditions (Design)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB: Moving CC: 228 CD: 228	RCS Temperature (°F): 547 Power Level (% F.P.): 0 Other (specify): Below Nuclear Heating
III Test Conditions (Actual)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB: Moving CC: 228 CD: 228	RCS Temperature (°F): 543.9 Power Level (% F.P.): 0 Other (Specify): Below Nuclear Heating
	Date/Time Test Performed: 5/28/83 1139	
IV  Test Results	Measured Parameter (Description)	$I_B^{REF}$ ; Integral Worth of Cntl Bank B, All Other Rods Out
	Measured Value	$I_B^{REF} = 1569 \text{ pcm}$
	Design Value (Actual Conditions)	$I_B^{REF} = 1544 \pm 154 \text{ pcm}$
	Design Value (Design Conditions)	$I_B^{REF} = 1544 \pm 154 \text{ pcm}$
	Reference	VEP-PSE-NFE-288
V Acceptance Criteria	FSAR/Tech Spec	If Design Tolerance is exceeded, SNSOC shall evaluate impact of test result on safety analysis. SNSOC may specify that additional testing be performed.
	Reference	VEP-FRD-36A
VI Comments	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	

Completed By: AS Thomas  
Test Engineer

Evaluated By: BD Mann

Recommended for  
Approval By: C. J. Snow  
NFO Engineer

NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description: Critical Boron Concentration - B Bank In Proc No /Section: 2-PT-94.3 Sequence Step No: 8	
II Test Conditions (Design)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB: 0 CC: 228 CD: 228	RCS Temperature (°F): 547 Power Level (% F.P.): 0 Other (specify): Below Nuclear Heating
III Test Conditions (Actual)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB: 0 CC: 228 CD: 228	RCS Temperature (°F): 543.8 Power Level (% F.P.): 0 Other (Specify): Below Nuclear Heating
	Date/Time Test Performed: 5/28/83 1655	
IV	Meas Parameter (Description)	$(C_B)_B^M$ ; Critical Boron Conc - B Bank In
Test Results	Measured Value	$(C_B)_B^M = 1375 \text{ ppm}$
	Design Value (Actual Cond)	$C_B = 1387 \pm 30 \text{ ppm}$
	Design Value (Design Cond)	$C_B = 1358 + \Delta C_B^{\text{Prev}} \pm (10 + 154.4/ \alpha_{C_B} ) \text{ ppm}$
	Reference	VEP-PSE-NFE-268
V	FSAR/Tech Spec	$\alpha_{C_B} \times C_B \leq 24,000 \text{ pcm}$
Acceptance Criteria	Reference	UFSAR Section 15.2.4
VI	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	
Comments	$\alpha_{C_B} = -7.94 \text{ pcm/ppm}$ for preliminary analysis $\Delta C_B^{\text{Prev}} = (C_B)_{\text{ARO}}^M - 1546$ $\alpha_{C_B} = -7.85 \text{ pcm/ppm}$ for final analysis.	

Completed By: B. Thomas  
Test Engineer

Evaluated By: B. O. Mann

Recommended for  
Approval By: C. J. Snow  
NFO Engineer

NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description: HZP Boron Worth Coefficient Measurement Proc No /Section: 2-PT-94.5 Sequence Step No: 7	
II Test Conditions (Design)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB: Moving CC: 228 CD: 228	RCS Temperature (°F): 547 Power Level (% F.P.): 0 Other (specify): Below Nuclear Heating
III Test Conditions (Actual)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB: Moving CC: 228 CD: 228	RCS Temperature (°F): 542.1 Power Level (% F.P.): 0 Other (Specify): Below Nuclear Heating
	Date/Time Test Performed: 5/28/83 0640	
IV  Test Results	Measured Parameter (Description)	$\alpha_{CB}$ , Boron Worth Coefficient
	Measured Value	$\alpha_{CB} = -7.85 \text{ pcm/ppm}$
	Design Value (Actual Conditions)	$\alpha_{CB} = -7.94 \pm 0.79 \text{ pcm/ppm}$
	Design Value (Design Conditions)	$\alpha_{CB} = -7.94 \pm 0.79 \text{ pcm/ppm}$
	Reference	VEP-PSE-NFE-288
V Acceptance Criteria	FSAR/Tech Spec	$\alpha_{CB} \times C_B \leq 24,000 \text{ pcm}$
	Reference	UFSAR Section 15.2.4
VI Comments	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	

Completed By: RS Thomas  
Test Engineer

Evaluated By: B.D. Mann

Recommended for  
Approval By: C. J. Snow  
NFO Engineer

NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description: Cntl Bank D Worth Measurement-Rod Swap Proc No /Section: 2-PT-94.7 Sequence Step No: 10	
II Test Conditions (Design)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB:Moving CC: 228 CD:Moving	RCS Temperature (°F): 547 Power Level (% F.P.): 0 Other (specify): Below Nuclear Heating
III Test Conditions (Actual)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB:Moving CC: 228 CD:Moving	RCS Temperature (°F): 543.7 Power Level (% F.P.): 0 Other (Specify): Below Nuclear Heating
IV Test Results	Date/Time Test Performed: 5/28/83 1737	
	Meas Parameter (Description)	$I_{D}^{RS}$ ; Int Worth of Cntl Bank D-Rod Swap
	Measured Value	$I_{D}^{RS} = 859 \text{ pcm}$ (Adj. Meas. Crit. Ref Bank Position = 106 steps)
	Design Value (Actual Cond)	$I_{D}^{RS} = 850 \pm 128 \text{ pcm}$ (Adj. Meas. Crit. Ref Bank Position = 106 steps)
	Design Value (Design Cond)	$I_{D}^{RS} = 851 \pm 128 \text{ pcm}$ (Critical Ref Bank Position = 106 steps)
	Reference	VEP-PSE-NFE-288, VEP-FRD-36A, NFO-TI-2.2A
V Acceptance Criteria	FSAR/Tech Spec	If Design Tolerance is exceeded, SNSOC shall evaluate impact of test result on safety analysis. SNSOC may specify that additional testing be performed.
	Reference	VEP-FRD-36A
VI Comments	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	

Completed By: OB Thomas  
Test Engineer

Evaluated By: Paul A. Raley

Recommended for  
Approval By : C. J. Snow  
NFO Engineer

NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description: Cntl Bank C Worth Measurement-Rod Swap Proc No /Section: 2-PT-94.7 Sequence Step No: 11	
II Test Conditions (Design)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB:Moving CC:Moving CD: 228	RCS Temperature (°F): 547 Power Level (% F.P.): 0 Other (specify): Below Nuclear Heating
III Test Conditions (Actual)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB:Moving CC:Moving CD: 228	RCS Temperature (°F): 543.4 Power Level (% F.P.): 0 Other (Specify): Below Nuclear Heating
	Date/Time Test Performed: 5/28/83 1837	
IV Test Results	Meas Parameter (Description)	$I_{C}^{RS}$ ; Int Worth of Cntl Bank C-Rod Swap
	Measured Value	$I_{C}^{RS} = 896 \text{ pcm}$ (Adj. Meas. Crit. Ref Bank Position = 110 steps)
	Design Value (Actual Cond)	$I_{C}^{RS} = 903 \pm 135 \text{ pcm}$ (Adj. Meas. Crit. Ref Bank Position = 110 steps)
	Design Value (Design Cond)	$I_{C}^{RS} = 900 \pm 135 \text{ pcm}$ (Critical Ref Bank Position = 111 steps)
	Reference	VEP-PSÉ-NFE-288, VEP-FRD-36A, NFO-TI-2.2A
V Acceptance Criteria	FSAR/Tech Spec	If Design Tolerance is exceeded, SNSOC shall evaluate impact of test result on safety analysis. SNSOC may specify that additional testing be performed.
	Reference	VEP-FRD-36A
VI Comments	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	

Completed By:

Q. Thomas  
Test Engineer

Evaluated by:

Paul J. Reed

Recommended for

Approval By :

C. J. Snow  
NFO Engineer



NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description: Cntl Bank A Worth Measurement-Rod Swap Proc No /Section: 2-PT-94.7 Sequence Step No: 12	
II Test Conditions (Design)	Bank Positions (Steps) SDA: 228 SDB: 228 CA:Moving CB:Moving CC: 228 CD: 228	RCS Temperature (°F): 547 Power Level (% F.P.): 0 Other (specify): Below Nuclear Heating
III Test Conditions (Actual)	Bank Positions (Steps) SDA: 228 SDB: 228 CA:Moving CB:Moving CC: 228 CD: 228	RCS Temperature (°F): 544 Power Level (% F.P.): 0 Other (Specify): Below Nuclear Heating
	Date/Time Test Performed: 5/20/83 1927	
IV Test Results	Meas Parameter (Description)	$I_{A}^{RS}$ ; Int Worth of Cntl Bank A - Rod Swap
	Measured Value	$I_{A}^{RS} = 758 \text{ pcm}$ (Adj. Meas. Crit. Ref Bank Position = 96 steps)
	Design Value (Actual Cond)	$I_{A}^{RS} = 667 \pm 100 \text{ pcm}$ (Adj. Meas. Crit. Ref Bank Position = 96 steps)
	Design Value (Design Cond)	$I_{A}^{RS} = 665 \pm 100 \text{ pcm}$ (Critical Ref Bank Position = 89 steps)
	Reference	VEP-PSE-NFE-288, VEP-FRD-36A, NFO-TI-2.2A
V Acceptance Criteria	FSAR/Tech Spec	If Design Tolerance is exceeded, SNSOC shall evaluate impact of test result on safety analysis. SNSOC may specify that additional testing be performed.
	Reference	VEP-FRD-36A
VI Comments	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	

Completed By: AS Thomas  
Test Engineer

Evaluated By: Paul J. Reddy

Recommended for  
Approval By: C. J. Snow  
NFO Engineer

NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description: Shutdown Bank B Worth Meas. - Rod Swap Proc No /Section: 2-PT-94.7 Sequence Step No: 13	
II Test Conditions (Design)	Bank Positions (Steps) SDA: 228 SDB:Moving CA: 228 CB:Moving CC: 228 CD: 228	RCS Temperature (°F): 547 Power Level (% F.P.): 0 Other (specify): Below Nuclear Heating
III Test Conditions (Actual)	Bank Positions (Steps) SDA: 228 SDB:Moving CA: 228 CB:Moving CC: 228 CD: 228	RCS Temperature (°F): 543.9 Power Level (% F.P.): 0 Other (Specify): Below Nuclear Heating
IV Test Results	Date/Time Test Performed: 5/28/83 2011	
	Meas Parameter (Description)	$I_{RS}$ $I_{SB}$ ; Int Worth of Shutdown Bank B-Rod Swap
	Measured Value	(Adj. Meas. Crit. Ref Bank $I_{RS} = 806 \text{ pcm}$ Position = 101 steps)
	Design Value (Actual Cond)	(Adj. Meas. Crit. Ref Bank $I_{RS} = 827 \pm 124 \text{ pcm}$ Position = 101 steps)
	Design Value (Design Cond)	$I_{RS} = 831 \pm 125 \text{ pcm}$ (Critical Ref Bank Position = 104 steps)
V Acceptance Criteria	Reference	VEP-PSE-NFE-288, VEP-FRD-36A, NFO-TI-2.2A
	FSAR/Tech Spec	If Design Tolerance is exceeded, SNSOC shall evaluate impact of test result on safety analysis. SNSOC may specify that additional testing be performed.
VI Comments	Reference	VEP-FRD-36A
	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	

Completed By: JB Thomas  
Test Engineer

Evaluated By: Paul J. Ralston

Recommended for  
Approval By : C. J. Snow  
NFO Engineer



NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description: Shutdown Bank A Worth Meas. - Rod Swap Proc No /Section: 2-PT-94.7 Sequence Step No: 14	
II Test Conditions (Design)	Bank Positions (Steps) SDA:Moving SDB: 228 CA: 228 CB:Moving CC: 228 CD: 228	RCS Temperature (°F): 547 Power Level (% F.P.): 0 Other (specify): Below Nuclear Heating
III Test Conditions (Actual)	Bank Positions (Steps) SDA:Moving SDB: 228 CA: 228 CB:Moving CC: 228 CD: 228	RCS Temperature (°F): 542.9 Power Level (% F.P.): 0 Other (Specify): Below Nuclear Heating
	Date/Time Test Performed: 5/28/83 2056	
IV Test Results	Meas Parameter (Description)	$I_{SA}^{RS}$ ; Int Worth of Shutdown Bank A-Rod Swap
	Measured Value	$I_{SA}^{RS} = 995 \text{ pcm}$ (Adj. Meas. Crit. Ref Bank Position = 121 steps)
	Design Value (Actual Cond)	$I_{SA}^{RS} = 945 \pm 142 \text{ pcm}$ (Adj. Meas. Crit. Ref Bank Position = 121 steps)
	Design Value (Design Cond)	$I_{SA}^{RS} = 947 \pm 142 \text{ pcm}$ (Critical Ref Bank Position = 116 steps)
	Reference	VEP-PSÉ-NFE-288, VEP-FRD-36A, NFO-TI-2.2A
V Acceptance Criteria	FSAR/Tech Spec	If Design Tolerance is exceeded, SNSOC shall evaluate impact of test result on safety analysis. SNSOC may specify that additional testing be performed.
	Reference	VEP-FRD-36A
VI Comments	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	

Completed By: CS Thomas  
Test Engineer

Evaluated By: Paul R. Kelly

Recommended for  
Approval By : C. J. Snow  
NFO Engineer

NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description: Total Rod Worth - Rod Swap Proc No /Section: 2-PT-94.7 Sequence Step No: 16	
II Test Conditions (Design)	Bank Positions (Steps) SDA:Moving SDB:Moving CA:Moving CB:Moving CC:Moving CD:Moving	RCS Temperature (°F): 547 Power Level (% F.P.): 0 Other (specify): Below Nuclear Heating
III Test Conditions (Actual)	Bank Positions (Steps) SDA:Moving SDB:Moving CA:Moving CB:Moving CC:Moving CD:Moving	RCS Temperature (°F): 543.7 Power Level (% F.P.): 0 Other (Specify): Below Nuclear Heating
	Date/Time Test Performed: 5/28/83 1737	
IV Test Results	Meas Parameter (Description)	I <sub>Total</sub> ; Int Worth of All Banks - Rod Swap
	Measured Value	I <sub>Total</sub> = 5883 pcm
	Design Value (Actual Cond)	I <sub>Total</sub> = 5736 ± 574 pcm
	Design Value (Design Cond)	I <sub>Total</sub> = 5738 ± 574 pcm
	Reference	VEP-PSE-NFE-288, VEP-FRD-36A, NFO-TI-2.2A
V Acceptance Criteria	FSAR/Tech Spec	If Design Tolerance is exceeded, SNSOC shall evaluate impact of test result on safety analysis. SNSOC may specify that additional testing be performed.
	Reference	VEP-FRD-36A
VI Comments	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	

Completed By: Q. S. Thomas  
Test Engineer

Evaluated By: Paul J. Ruff

Recommended for  
Approval By: C. J. Snow  
NFO Engineer

NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description : M/D Flux Map - HZP, ARO Proc No / Section: 2-PT-21.1 Sequence Step No: 6				
II Test Conditions (Design)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB : 228 CC : 228 CD: 228		RCS Temperature (°F): $T_{REF} \pm 1$ Power Level (% F.P.): ~ 3 Other (specify) Must have $\geq 38$ thimbles		
III Test Conditions (Actual)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB : 228 CC : 228 CD: 211		RCS Temperature (°F): $T_{REF}$ Power Level (% F.P.): 4 Other (Specify): 41 Thimbles		
	Date/Time Test: Performed: 5/29/83 0512				
IV	Meas Parameter (Description)	MAX. REL ASSY PWR % DIFF (M-P)/P	NUC ENTHAL RISE HOT CHAN FACT F-dH(N)	TOTAL HEAT FLUX HOT CHAN FACT -Q(T)	QUADRANT POWER TILT RATIO QPTR
Test Results	Measured Value	8.3% for $P_{11} = 1.18$ 7.9% for $P_{11} = 0.62$	1.574	2.479	1.028*
	Design Value (Design Conds)	8.1% for $P_{11} = 0.9$ 8.1% for $P_{11} = 0.9$ ( $P_{11} = \text{design. Pwr.}$ )	$P_{11} \leq 1.25(1 + 3(1-P))$ [1-CBP(N)]	$P_{11}(T) \leq 4.40 \text{ in Hg(T)}$	$\leq 1.02$
	Reference	WCAP-7905 REV.1	NONE	NONE	WCAP-7905 REV.1
V Acceptance Criteria	FSAR/Tech Spec	NONE	NA	NA	NA
	Reference	NONE	TS 3.2.3	TS 3.2.2	TS 3.2.4
VI Comments	Design Tolerance is met : <input type="checkbox"/> YES <input checked="" type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO # Design tolerance not met but test results are acceptable per North Anna Power Station Deviation Report B3-477.				

Completed By:

Q. Thomas  
Test Engineer

Evaluated By:

M. J. Pierce

Recommended for

Approval By :

C. J. Snow  
NFO Engineer

NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description : M/D Flux Map-At Power, NI Calibration Proc No / Section: 2-PT-22.2 Sequence Step No: 43				
II Test Conditions (Design)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB : 228 CC : 228 CD: *		RCS Temperature (°F): $T_{REF} \pm 1$ Power Level (% F.P.): ~ AR Other (specify) Must have $\geq 38$ thimbles		
III Test Conditions (Actual)	Bank Positions (Steps) SDA: 228 SDB: 228 CA: 228 CB : 228 CC : 228 CD: 179		RCS Temperature (°F): $T_{REF}$ Power Level (% F.P.): 49.7% Other (Specify): 45 thimbles		
IV   Test Results	Date/Time Test: Performed: 6/06/83 0823				
	Meas Parameter (Description)	MAX. REL ASSY PWR % DIFF (M-P)/P	NUC ENTHAL RISE HOT CHAN FACT F-dH(N)	TOTAL HEAT FLUX HOT CHAN FACT F-Q(T)	QUADRANT POWER TILT RATIO QPTR
	Measured Value	11.1% for $P_1 = 1.11$ 12.4% for $P_2 = 0.15$	1.545	2.139	1.015
	Design Value (Design Conds)	$\pm 10\%$ for $P_1 \geq 0.9$ $\pm 15\%$ for $P_1 < 0.9$ ( $P_1$ = Assy. Pwr.)	NA	NA	$\leq 1.02$
	Reference	WCAP-7905 REV.1	NONE	NONE	WCAP-7905 REV.1
V Acceptance Criteria	FSAR/Tech Spec	NONE	$P_{AS}^{(1)} \leq 1.22(1 + 2(1-P_1))$ (1 = 0.001 (100))	$P_0^{(2)} \leq 4.40 \pm 2(2)$ for $P \leq 0.3$ $P_0^{(2)} \leq 2.30/P \pm 2(2)$ for $P > 0.3$	NA
	Reference	NONE	TS 3.2.3	TS 3.2.2	TS 3.2.4
VI Comments	Design Tolerance is met : <input type="checkbox"/> YES <input checked="" type="checkbox"/> NO				
	Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO				
* As Required A violation of the FSX Technical Specifications limit occurred, but was deemed acceptable as per North Anna Power Station Deviation Report #83-519.					

Completed By: RT Leonard  
Test Engineer

Evaluated By: BD Mann

Recommended for  
Approval By: C. J. Snow  
NFO Engineer

1. This deviation is deemed acceptable as per North Anna  
Power Station Deviation Report #83-551.

NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description : M/D Flux Map-At Power, NI Calibration Proc No / Section: 2-PT-22.2 Sequence Step No: 44				
II Test Conditions (Design)	Bank Positions (Steps)		RCS Temperature (°F): $T_{REF} \pm 1$ Power Level (% F.P.): ~ AR Other (specify)		
	SDA: 228 SDB: 228 CA: 228 CB: 228 CC: 228 CD: *				
III Test Conditions (Actual)	Bank Positions (Steps)		RCS Temperature (°F): $T_{REF}$ Power Level (% F.P.): 49.8% Other (Specify): 21 thimbles		
	SDA: 228 SDB: 228 CA: 228 CB: 228 CC: 228 CD: 161		Quarter-Core Flux Map		
IV	Date/Time Test: Performed: 6/06/83 1600				
	Meas Parameter (Description)	MAX. REL ASSY PWR % DIFF (M-P)/P	NUC ENTHAL RISE HOT CHAN FACT F-dH(N)	TOTAL HEAT FLUX HOT CHAN FACT F-Q(T)	QUADRANT POWER TILT RATIO QPTR
Test Results	Measured Value	5.7% $P_1 = 1.06$ $P_2 = 0.52$ $P_3 = 1.06$ $P_4 = 0.52$	NA <sup>2</sup>	NA <sup>2</sup>	NA <sup>2</sup>
	Design Value (Design Conds)	8.10% for $P_1 \leq 0.9$ 8.10% for $P_1 \leq 0.9$ ( $P_1$ = Assy. Pwr.)	NA	NA	≤ 1.02
	Reference	WCAP-7905 REV.1	NONE	NONE	WCAP-7905 REV.1
V Acceptance Criteria	FSAR/Tech Spec	NONE	$P_{max} \leq 1.33(1 + 3(1-P_1))$ (1 = Assy. Pwr.)	$P_Q(2) \leq 2.20/P_1 = K(2)$	NA
	Reference	NONE	TS 3.2.3	TS 3.2.2	TS 3.2.4
VI Comments	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Must have at least 38 thimbles for a full-core flux map or, at least 16 thimbles for a quarter-core flux map. * As Required				

Completed By: BSThoras  
Test Engineer

Evaluated By: BD Mann

Recommended for  
Approval By: C. J. Snow  
NFO Engineer

1. These parameters are not verified using a partial-core map  
obtained for NI calibration.



NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description : M/D Flux Map-At Power, NI Calibration Proc No / Section: 2-PT-22.2 Sequence Step No: 45				
II Test Conditions (Design)	Bank Positions (Steps)		RCS Temperature (°F): $T_{REF} \pm 1$ Power Level (% F.P.): ~ AR Other (specify)		
	SDA: 228 SDB: 228 CA: 228 CB : 228 CC : 228 CD: *				
III Test Conditions (Actual)	Bank Positions (Steps)		RCS Temperature (°F): $T_{REF}$ Power Level (% F.P.): 77.2 % Other (Specify): 50 thimbles		
	Date/Time Test: Performed: 6/07/83 0606				
IV Test Results	Meas Parameter (Description)	MAX. REL ASSY PWR % DIFF (M-P)/P	NUC ENTHAL RISE HOT CHAN FACT F-dH(N)	TOTAL HEAT FLUX HOT CHAN FACT F-Q(T)	QUADRANT POWER TILT RATIO QPTR
	Measured Value	8.37% for $P = 1.06$ 9.57% for $P = 0.17$	1.480	1.980	1.011
	Design Value (Design Conds)	± 10% for $P_1 \geq 0.9$ ± 15% for $P_1 < 0.9$ ( $P_1$ = Assy. Pwr.)	NA	NA	≤ 1.02
	Reference	WCAP-7905 REV.1	NONE	NONE	WCAP-7905 REV.1
V Acceptance Criteria	FSAR/Tech Spec	NONE	$P_{avg} \leq 1.15 (1 + 3(1-P))$ (1-WPP(10))	$P_{avg} \leq 2.20/P + 2(2)$	NA
	Reference	NONE	TS 3.2.3	TS 3.2.2	TS 3.2.4
VI Comments	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO Must have at least 38 thimbles for a full-core flux map, or at least 16 thimbles for a quarter-core flux map. * As Required				

Completed By: Q. Thomas  
Test Engineer

Evaluated By: B.D. Mann

Recommended for  
Approval By: C. J. Snow  
NFO Engineer

NORTH ANNA POWER STATION UNIT 2 CYCLE 3  
STARTUP PHYSICS TEST RESULTS AND EVALUATION SHEET

I Reference	Test Description : M/D Flux Map - HFP, ARO, Eq. Xe Proc No / Section: 2-PT-21.1 Sequence Step No: 46				
II Test Conditions (Design)	Bank Positions (Steps)		RCS Temperature (°F): $T_{REF} \pm 1$		
	SDA: 228 SDB: 228 CA: 228 CB: 228 CC: 228 CD: *		Power Level (% F.P.): $95 \pm 5$ Other (specify): Eq. Xe. Must have $\geq 38$ thimbles		
III Test Conditions (Actual)	Bank Positions (Steps)		RCS Temperature (°F): $T_{REF}$		
	SDA: 228 SDB: 228 CA: 228 CB: 228 CC: 228 CD: 228		Power Level (% F.P.): 100% Other (Specify): 49 thimbles		
IV  Test Results	Date/Time Test: Performed: 6/10/83 0749				
	Meas Parameter (Description)	MAX. REL ASSY PWR % DIFF (M-P)/P	NUC ENTHAL RISE HOT CHAN FACT F-dH(N)	TOTAL HEAT FLUX HOT CHAN FACT F-Q(T)	QUADRANT POWER TILT RATIO QPTR
	Measured Value	2.8% for $P_1 = 1.07$ 7.1% for $P_1 = 0.10$	1.473	1.906	1.01
	Design Value (Design Conds)	± 10% for $P_1 \leq 0.5$ ± 15% for $P_1 > 0.5$ ( $P_1$ = Assy. Pwr.)	NA	NA	≤ 1.02
	Reference	WCAP-7905 REV.1	NONE	NONE	WCAP-7905 REV.1
V Acceptance Criteria	FSAR/Tech Spec	NONE	$P_{AS} \leq \frac{1.33(1 + 3(1-P))}{(1-0.8P)(W)}$	$P_Q(Z) \leq 2.20/P \times R(Z)$	NA
	Reference	NONE	TS 3.2.3	TS 3.2.2	TS 3.2.4
VI Comments	Design Tolerance is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO				
	Acceptance Criteria is met : <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO				
* As Required					

Completed By: RT Thomas  
Test Engineer

Evaluated By: BD Smith

Recommended for  
Approval By: C. J. Snow  
NFO Engineer



VIRGINIA ELECTRIC AND POWER COMPANY  
RICHMOND, VIRGINIA 23261

July 19, 1983

W. L. STEWART  
VICE PRESIDENT  
NUCLEAR OPERATIONS

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
ATTN: Mr. Robert A. Clark, Chief  
Operating Reactors Branch No. 3  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Serial No. 410  
NOD:BDM:hca

Docket No. 50-339  
License No. NPF-7

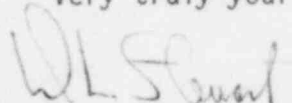
Gentlemen:

NORTH ANNA POWER STATION  
UNIT 2, CYCLE 3 STARTUP PHYSICS TEST REPORT

For your information, enclosed are five copies of the Vepco Topical Report VEP-NOS-4, "North Anna Unit 2, Cycle 3 Startup Physics Test Report".

Should you have any questions, please contact us.

Very truly yours

  
W. L. Stewart

Enclosures

cc: Mr. James P. O'Reilly  
Regional Administrator  
Region II

Mr. M. B. Shymlock  
NRC Resident Inspector  
North Anna Power Station

IE26  
1/5