

U.S. NUCLEAR REGULATORY COMMISSION  
LICENSEE EVENT REPORT

APPROVED BY OMB  
3150-0011  
EXPIRES 4-30-82

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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CON'T  
01 REPORT SOURCE I 050000395 06 1083 07 070883 08

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES  
On June 10, 1983, the NRC Resident Inspector identified that the surveillance requirements for the six-inch containment purge supply and exhaust isolation valves were not being fully conducted. The opening and closing times for the valves were being logged, but the cumulative run time for the previous 365 days was not being tracked per the applicable technical specification. No adverse consequences resulted as the 1000 hour limit on opening the valves had not been exceeded.

09 SYSTEM CODE SC 11 CAUSE CODE D 12 CAUSE SUBCODE Z 13 COMPONENT CODE VALVEX 14 COMP. SUBCODE E 15 VALVE SUBCODE D 16  
17 LER/RO REPORT NUMBER 83 18 ACTION TAKEN 19 FUTURE ACTION 20 EFFECT ON PLANT 21 SHUTDOWN METHOD 22 HOURS 23 OCCURRENCE CODE 03 24 REPORT TYPE L 25 REVISION NO. 0  
26 ACTION TAKEN 27 FUTURE ACTION 28 EFFECT ON PLANT 29 SHUTDOWN METHOD 30 HOURS 31 OCCURRENCE CODE 32 REPORT TYPE 33 REVISION NO.  
34 ACTION TAKEN 35 FUTURE ACTION 36 EFFECT ON PLANT 37 SHUTDOWN METHOD 38 HOURS 39 OCCURRENCE CODE 40 REPORT TYPE 41 REVISION NO.  
42 ACTION TAKEN 43 FUTURE ACTION 44 EFFECT ON PLANT 45 SHUTDOWN METHOD 46 HOURS 47 OCCURRENCE CODE 48 REPORT TYPE 49 REVISION NO.

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS  
The event is attributed to procedural inadequacy. The requirements of the technical specification were not fully addressed. A surveillance test procedure has been developed to address the technical specification surveillance requirements of the subject valves. This procedure was issued on July 1, 1983. No further action is planned.

15 FACILITY STATUS E 28 % POWER 092 29 OTHER STATUS N/A 30 METHOD OF DISCOVERY D 31 DISCOVERY DESCRIPTION NRC Resident Inspector 32

16 ACTIVITY CONTENT RELEASED OF RELEASE 2 33 2 34 AMOUNT OF ACTIVITY N/A 35 LOCATION OF RELEASE N/A 36

17 PERSONNEL EXPOSURES NUMBER 000 37 TYPE 2 38 DESCRIPTION N/A 39

18 PERSONNEL INJURIES NUMBER 000 40 DESCRIPTION N/A 41

19 LOSS OF OR DAMAGE TO FACILITY TYPE 2 42 DESCRIPTION N/A 43

20 PUBLICITY ISSUED N 44 DESCRIPTION N, S 45  
8307220454 830708  
PDR ADOCK 05000395  
PDR

NAME OF PREPARER

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83 JUL 13 AIO: 35

O. W. DIXON, JR.  
VICE PRESIDENT  
NUCLEAR OPERATIONS

July 8, 1983

Mr. James P. O'Reilly  
Regional Administrator  
U.S. Nuclear Regulatory Commission  
Region II, Suite 2900  
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Atlanta, Georgia 30303

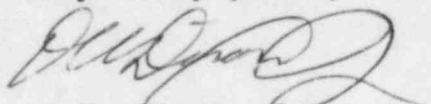
SUBJECT: Virgil C. Summer Nuclear Station  
Docket No. 50/395  
Operating License No. NPF-12  
Thirty Day Written Report  
LER 83-061

Dear Mr. O'Reilly:

Please find attached Licensee Event Report #83-061 for Virgil C. Summer Nuclear Station. This Thirty Day Report is required by Technical Specification 6.9.1.13.(c) as a result of the failure to satisfy the Surveillance Requirements of Technical Specification 4.6.1.7.2, "Containment Ventilation System," on June 10, 1983.

Should there be any questions, please call us at your convenience.

Very truly yours,

  
O. W. Dixon, Jr.

HCF:OWD/mac  
Attachment

cc: V. C. Summer  
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Mr. James P. O'Reilly  
LER No. 83-061  
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July 8, 1983

#### EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

On June 10, 1983, with the Plant in Mode 1, the Nuclear Regulatory Commission Resident Inspector identified that the surveillance requirements for the six-inch containment purge supply and exhaust isolation valves were not being properly performed in accordance with Technical Specification 4.6.1.7.2. This Technical Specification requires that the cumulative time that the six-inch purge supply and exhaust isolation valves have been open during the previous 365 days be determined at least once per seven (7) days.

The opening and closing times of the valves were being documented on the Control Building logs; however, the cumulative time that the valves had been open during the previous 365 days were not determined.

There were no adverse consequences resulting from the event as the Technical Specification limit that the six-inch purge supply and exhaust isolation valves be open for less than 1000 hours per 365 days had not been exceeded.

#### CAUSE AND CORRECTIVE ACTIONS

The cause of the event is attributed to procedural inadequacy in that the surveillance requirements of Technical Specification 4.6.1.7.2 were not fully addressed.

A weekly surveillance test procedure has been developed to implement the surveillance requirements of Technical Specification 4.6.1.7.2. This procedure was issued July 1, 1983.

The licensee considers this event to be an isolated case and plans no further action.