

## LICENSEE EVENT REPORT

Update Report:

Previous Report Date 5-4-83

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CONTROL BLOCK: 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

0 1 N C B E P 1 2 0 0 - 0 0 0 0 0 0 - 0 0 0 3 4 1 1 1 1 1 4 5  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CON'T

0 1 REPORT SOURCE L 6 0 5 0 - 0 3 2 5 7 0 4 0 6 8 3 8 0 7 1 1 2 8 3 9  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During ongoing Unit No. 1 refueling operations on April 6 and 8 of 1983, the

0 3 following control rods had no indications for the identified positions: rod 46-35

0 4 ("00"), rod 22-19 (7, 17, 27, 37, 47); rod 14-11 (8, 18, 28, 38, 48, "full-out"); rod

0 5 42-31 ("full in") rod 46-31 ("full-out"). In each case, the health and safety of the

0 6 public was not affected.

0 7

0 8 Technical Specifications 3.1.3.7, 6.9.1.9b

0 9 SYSTEM CODE I B 11 CAUSE CODE X 12 CAUSE SUBCODE Z 13 COMPONENT CODE X X X X X X X 14 COMP. SUBCODE Z 15 VALVE SUBCODE Z 16

17 LER/RO REPORT NUMBER 8 3 21 22 EVENT YEAR 8 3 21 22 SEQUENTIAL REPORT NO. 0 1 4 24 26 OCCURRENCE CODE 0 3 28 29 REPORT TYPE L 30 31 REVISION NO. 1 32

ACTION TAKEN X 18 FUTURE ACTION F 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0 0 0 0 22 ATTACHMENT SUBMITTED Y 23 NPRD-4 FORM SUB. Y 24 PRIME COMP. SUPPLIER N 25 COMPONENT MANUFACTURER G 0 8 0 0 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Loose or defective PIP connectors in rods 46-35, 22-19, and 14-11, a defective reed

1 1 switch or connector pin in rod 46-31, and a defective PIP probe in rod 42-31 caused

1 2 each subject indication problem. The indication problems affecting these rods

1 3 will be repaired prior to the completion of the ongoing refueling outage.

1 4

1 5 FACILITY STATUS H 28 % POWER 0 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY A 31 DISCOVERY DESCRIPTION Operator Surveillance 32

1 6 ACTIVITY CONTENT Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36

1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39

1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43

2 0 PUBLICITY ISSUED N 44 DESCRIPTION NA 45

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PDR AD0CK 05000325  
S PDR

NRC USE ONLY

NAME OF PREPARER M. J. Pastva, Jr.

PHONE 919-457-9521

LER ATTACHMENT - RO #1-83-14

Facility: Unit No. 1

Event Date: April 6, 1983

Plant modifications to install shroud-like protectors/supportive devices around the PIP connectors of control rods on Unit Nos. 1 and 2 are respectively scheduled for installation prior to the completion of each unit's next refueling outage. It is felt these modifications will help reduce the number of control rod PIP connector failures which result from inadvertent bumping during undervessel work.

USNRC REGION II  
ATLANTA, GEORGIA  
**CP&L**

83 JUL 14 AM: 08  
Carolina Power & Light Company

Brunswick Steam Electric Plant  
P. O. Box 10429  
Southport, NC 28461-0429

July 12, 1983

FILE: B09-13510C  
SERIAL: BSEP/83-2126

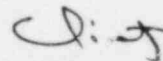
Mr. James P. O'Reilly, Administrator  
U. S. Nuclear Regulatory Commission  
Region II, Suite 3100  
101 Marietta Street NW  
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1  
DOCKET NO. 50-325  
LICENSE NO. DPR-71  
SUPPLEMENT TO LICENSEE EVENT REPORT 1-83-14

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 1, the enclosed supplemental Licensee Event Report is submitted. The original report fulfilled the requirement for a written report within thirty (30) days of a reportable occurrence and both are in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,



C. R. Dietz, General Manager  
Brunswick Steam Electric Plant

RMP/shb/LETSB1

Enclosure

cc: Mr. R. C. DeYoung  
NRC Document Control Desk

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11