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VPNPD-94-124  
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10 CFR 50.4  
10 CFR 50.90

November 16, 1994

Document Control Desk  
U.S. NUCLEAR REGULATORY COMMISSION  
Mail Station P1-137  
Washington, DC 20555

Gentlemen:

DOCKETS 50-266 AND 50-301  
ADDENDUM TO TECHNICAL SPECIFICATION CHANGE REQUEST 169  
MODIFICATIONS TO TECHNICAL SPECIFICATION 15.3.3  
"EMERGENCY CORE COOLING SYSTEM, AUXILIARY COOLING SYSTEMS,  
AIR RECIRCULATION FAN COOLERS, AND CONTAINMENT SPRAY"  
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

On September 12, 1994, we submitted Technical Specifications Change Request (TSCR) 169, "Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray," for Point Beach Nuclear Power Plant (PBNP) Units 1 and 2. The proposed changes modify Technical Specification 15.3.3, "Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray" to incorporate allowed outage times similar to those contained in NUREG 1431, Revision 0, "Westinghouse Owner's Group Improved Standard Technical Specifications," and clarify the operability requirements for the service water pumps. Proposed changes were also included to clarify the completion times for placing a unit in hot or cold shutdown if a limiting condition for operation cannot be met.

Our proposed change to Technical Specification 15.3.3.A.3 required that the reactor be placed in hot shutdown within 6 hours and reactor coolant system temperatures be maintained between 500°F and 350°F if residual heat removal (RHR) components are not returned to service within the required time period. Per discussion with Mr. Allen Hansen of your staff, we are hereby submitting a modification to our proposed change to state that reactor coolant system temperatures shall be maintained greater than 350°F after placing the reactor in the hot shutdown condition.

We are also modifying our September 12, 1994, submittal to remove the note to Specification 15.3.3.D.1.a. This note discussed the power supply to Train B service water pumps for the Unit 2, 1994, refueling outage and is no longer applicable.

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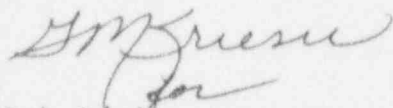
Document Control  
November 16, 1994  
Page 2

We have reviewed the safety evaluation and no significant hazards consideration included in our September 12, 1994, submittal and have determined that they apply to this addendum.

Copies of the affected Technical Specifications pages are enclosed.

Please contact us if you have any questions.

Sincerely,



Bob Link  
Vice President  
Nuclear Power

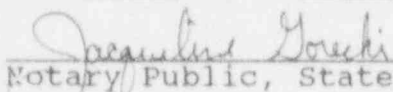
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Enclosure

cc: NRC Resident Inspector  
NRC Regional Administrator, Region III

Subscribed and sworn before me on

this 16<sup>th</sup> day of November 1994.

  
Notary Public, State of Wisconsin

My commission expires 10/27/96.