

Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

November 15, 1994
LIC-94-0237

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

SUBJECT: October 1994 Monthly Operating Report (MOR)

Enclosed please find the October, 1994 MOR for Fort Calhoun Station (FCS)
Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,



W. G. Gates
Vice President

WGG/d11

Enclosures

c: LeBoeuf, Lamb, Greene & MacRae
L. J. Callan, NRC Regional Administrator, Region IV
S. D. Bloom, NRC Project Manager
R. P. Mullikin, NRC Senior Resident Inspector
R. T. Pearce, Combustion Engineering
R. J. Simon, Westinghouse
INPO Records Center

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

OCTOBER 1994
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of October, Fort Calhoun Station (FCS) operated at a nominal 100% power, except for a planned short term power reduction to characterize a potential fuel pin leak. FCS reduced power to 32% on October 21, 1994, and returned to 100% power on October 26. Normal plant maintenance, surveillance, equipment rotation activities and online modification activities also occurred during the month.

The Reactor Protective System "B" Channel Trip Unit No. 10 (Loss of Load) was declared inoperable on October 6 for Instrumentation & Control (I&C) troubleshooting due to spurious pretrips. A 48-hour Limiting Condition for Operation, as required by Technical Specification 2.15.1, was entered and subsequently exited on October 7 after repairs were completed.

On October 7, a one-hour report was made to the NRC pursuant to 10 CFR 50.72(b)(1)(ii)(B) and 10 CFR 50.72(b)(2)(iii)(B). A review of the raw water pumps' design basis determined that seal water is essential for pump operation. However, the current seal water system is not classified as critical quality element (CQE) equipment. Interim measures were implemented to ensure that the raw water pumps remained operable. A modification is being processed to provide final resolution of this issue.

On October 10, a switch from morpholine to ethanolamine (ETA) was implemented for secondary chemistry control. The reason for this switch was to further reduce feed water iron concentrations.

On October 25, Chemistry sampling results determined that the particulate concentration exceeded the FCS administrative limits for the fuel oil in the Diesel Generator (DG) Storage Tank FO-1 and the DG-2 Day Tank. A filter unit was placed into service on FO-1 and chemistry returned to specification on October 28. Filtering work continues on the DG-2 Day Tank.

The following NRC inspection was completed during this reporting period:

<u>IER No.</u>	<u>Description</u>
94-21	Monthly Resident Inspection

There were no Licensee Event Reports submitted during this reporting period.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of October, no PORV or primary safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

The Reactor Coolant System leak rate was steady throughout the month with the following two exceptions. First, on October 21, FCS reduced power (to characterize a potential fuel pin leak) which resulted in five days of variable leak rates because of transient conditions while decreasing and increasing power. Second, an increase in unknown leakage on October 28 was caused by packing leakage on a charging pump. With this charging pump isolated for repacking, the leak rate returned to a nominal .10 gpm, which was the rate of leakage noted during most of the month, with no degrading trends noted. The leak rate has remained relatively unchanged during this cycle.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

<u>Amendment No.</u>	<u>Description</u>
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None	
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5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF OCTOBER 1994

- Reinject the Furmanite enclosures for the Main Steam Bypass Valves, HCV-1041C and HCV-1042C, due to leakage.
- Replaced the breaker for the Pressurizer Back-up Heaters Bank #1, Group #2, MCC-3A1-C01, due to failure.
- Repositioned the discs, replaced the solenoid valves and remounted the actuator air regulators on the Component Cooling Water outlet and inlet valves, HCV-2895A and HCV-2895B, for the Vacuum Deaerator Pumps DW-46A and DW-46B.
- Removed and inspected the alternate Raw Water Pump Seal Water supply piping on the four Raw Water Pumps, AC-10A, AC-10B, AC-10C and AC-10D.

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE NOVEMBER 07, 1994
COMPLETED BY D. L. LIPPY
TELEPHONE (402) 533-6843

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: OCTOBER 1994

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
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11. Hours in Reporting Period.....	745.0	7296.0	184970.0
12. Number of Hours Reactor was Critical	745.0	7262.2	144953.9
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	745.0	7247.1	143310.3
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1057928.2	10676682.7	189962482.3
17. Gross Elec. Energy Generated (MWH)..	355564.0	3573712.0	62661736.2
18. Net Elec. Energy Generated (MWH)....	338792.3	3408557.9	59781620.8
19. Unit Service Factor.....	100.0	99.3	77.5
20. Unit Availability Factor.....	100.0	99.3	77.5
21. Unit Capacity Factor (using MDC Net)	95.1	97.7	70.0
22. Unit Capacity Factor (using DER Net)	95.1	97.7	68.3
23. Unit Forced Outage Rate.....	.0	.7	4.0

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
THE 1995 REFUELING OUTAGE IS SCHEDULED TO COMMENCE MARCH 11, 1995, WITH
A PLANNED DURATION OF 49 DAYS.

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.): Forecast Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

N/A

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	NOVEMBER 07, 1994
COMPLETED BY	D. L. LIPPY
TELEPHONE	(402) 533-6843

MONTH OCTOBER 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	479	17	482
2	479	18	481
3	480	19	481
4	480	20	481
5	481	21	471
6	480	22	188
7	481	23	124
8	481	24	315
9	482	25	472
10	482	26	481
11	482	27	484
12	482	28	484
13	482	29	484
14	482	30	484
15	482	31	484
16	482		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE November 7, 1994
COMPLETED BY D. L. Lippy
TELEPHONE (402) 533-6843

REPORT MONTH October 1994

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
94-04	941021	S	0	H	4	N/A	ZZ	ZZZZZZ	During October, the plant operated at a nominal 100% power, with the exception of a planned short term power reduction to 32% power on October 21 to characterize a potential fuel pin leak. Power was returned to 100% on October 26.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-D161)

5
Exhibit H - Same Source

Attachment IV
Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending October 31, 1994

1. Scheduled date for next refueling shutdown. March 11, 1995
2. Scheduled date for restart following refueling. April 29, 1995
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? No
 - a. If answer is yes, what, in general, will these be? N/A
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. No
 - c. If no such review has taken place, when is it scheduled? Prior to April 1995
4. Scheduled date(s) for submitting proposed licensing action and support information. No submittal planned
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. *
6. The number of fuel assemblies:
 - a) in the core 133 Assemblies
 - b) in the spent fuel pool 570 Assemblies
 - c) spent fuel pool storage capacity 1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 2007 Outage

* OPPD is utilizing the CASMO-3/SIMULATE-3 codes for reactor physics related analyses for Cycle 16.

Prepared by Mark [Signature] for KCH Date 11/8/94