

ENCLOSURE 5

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
NRC DOCKET NOS. 50-325 AND 50-324
OPERATING LICENSE NOS. DPR-71 AND DPR-62
REQUEST FOR LICENSE AMENDMENTS
CONTAINMENT INTEGRATED LEAKAGE RATE TESTING
MARKED-UP TECHNICAL SPECIFICATION PAGES - UNIT 1

CONTAINMENT SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION (Continued)

- c. The leakage rate to less than or equal to 11.5 scf per hour for any one main steam line isolation valve,

prior to increasing reactor coolant system temperature above 212°F.

SURVEILLANCE REQUIREMENTS

in accordance with the

4.6.1.2 The primary containment leakage rates shall be demonstrated ~~at the following schedule and shall be determined in conformance with the criteria specified in Appendix J of 10CFR50, Appendix J, as modified by approved exemptions.~~ The provisions of Technical Specification 4.0.2 are not applicable to the test intervals specified in

- a. ~~Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at 40 ± 10 month intervals during shutdown at P₁, 49 psig, or P₂, 25 psig, during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection. (Deleted.)~~ ^{10 CFR 50, Appendix J.}
- b. ~~If any periodic Type A test fails to meet either 0.75 L₀ or 0.75 L₁, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet 0.75 L₀ or 0.75 L₁, a Type A test shall be performed at each plant shutdown for refueling or every 18 months, whichever occurs first, until two consecutive Type A tests meet 0.75 L₀ or 0.75 L₁, at which time the above test schedule may be resumed. (Deleted.)~~
- c. ~~The accuracy of each Type A test shall be verified by a supplemental test which: (Deleted.)~~
- ~~1. Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within 0.25 L₀ or 0.25 L₁.~~
 - ~~2. Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.~~
 - ~~3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage at P₁, 49 psig or P₂, 25 psig.~~

(Pages 3/4 6-3A and 3/4 6-3B have been deleted.)

A one time exemption from the accelerated testing requirements of Specification 4.6.1.2.b has been granted. The next Unit 1 Type A test will be performed during the Reload 9 outage after which two consecutive Type A test failures are required to resume the accelerated test schedule.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

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- d. Type B and C tests shall be conducted with gas at P_a , 49 psig at intervals no greater than 24 months except for tests involving:
 - 1. Air locks,
 - 2. Main steam line isolation valves,
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. Main steam line isolation valves shall be leak tested at least once per 18 months.
- g. All test leakage rates shall be calculated using observed data, converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.
- h. The provisions of Specification 4.0.2 are not applicable to 24 month and 40 ± 10 month surveillance intervals.

*The end of the current surveillance period for the surveillance requirements specified below may be extended beyond the time limit specified by Technical Specification 4.0.2. After April 1, 1985, the plant shall not be operated in Operational Conditions 1, 2, or 3 until the surveillance requirements listed below have been completed. Upon accomplishment of the surveillances, the provisions of Technical Specification 4.0.2 shall apply.

Technical Specification 4.6.1.2.d, as applicable to the penetrations and valves which correspond to the Test Nos. listed in Table 4.6.1.2-1.

**The end of the current surveillance period for the surveillance requirements specified below may be extended beyond the time limit specified by Technical Specification 4.0.2a. After April 1, 1985, the plant shall not be operated in Operational Conditions 1, 2, or 3 until the surveillance requirements listed below have been completed. Upon accomplishment of the surveillances, the provisions of Technical Specification 4.0.2a shall apply.

Technical Specification 4.6.1.2.f; as applicable to main steam isolation valves B21-FO22A, B21-FO22B, B21-FO22C, B21-FO22D, B21-FO28A, B21-FO28B, B21-FO28C, and B21-FO28D.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

TABLE 4.6.1.2-1

The following Test Numbers correspond to those listed in Attachment 1 of Carolina Power & Light Company's letter Serial: NLS-84-371 dated September 4, 1984.

| | |
|--------|--------|
| CAC-1 | E11-31 |
| CAC-2 | E11-32 |
| CAC-9 | E21-1 |
| CAC-10 | E21-2 |
| CAC-11 | E21-3 |
| CAC-12 | E21-4 |
| CAC-14 | E21-5 |
| CAC-15 | E21-7 |
| CAC-16 | E21-8 |
| CAC-17 | E41-2 |
| CAC-18 | E41-4 |
| CAC-19 | E41-6 |
| CAC-20 | E41-8 |
| CAC-21 | E51-2 |
| CAC-22 | E51-3 |
| CAC-23 | E51-4 |
| CAC-24 | SA-1 |
| CAC-25 | TIP-1 |
| CAC-26 | TIP-2 |
| CAC-39 | TIP-3 |
| CAC-40 | TIP-4 |
| CAC-41 | TIP-5 |
| CAC-42 | |
| E11-1 | |
| E11-2 | |
| E11-4 | |
| E11-6 | |
| E11-11 | |
| E11-12 | |
| E11-15 | |
| E11-16 | |
| E11-17 | |
| E11-19 | |
| E11-20 | |
| E11-22 | |
| E11-24 | |
| E11-26 | |
| E11-27 | |
| CAC-43 | |
| E11-28 | |
| E11-29 | |
| E11-30 | |

DELETED

ENCLOSURE 6

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
NRC DOCKET NOS. 50-325 AND 50-324
OPERATING LICENSE NOS. DPR-71 AND DPR-62
REQUEST FOR LICENSE AMENDMENTS
CONTAINMENT INTEGRATED LEAKAGE RATE TESTING
MARKED-UP TECHNICAL SPECIFICATION PAGES - UNIT 2

CONTAINMENT SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION (Continued)

- c. The leakage rate to less than or equal to 11.5 scf per hour for any one main steam line isolation valve,

prior to increasing reactor coolant system temperature above 212°F.

SURVEILLANCE REQUIREMENTS

in accordance with the

4.6.1.2 The primary containment leakage rates shall be demonstrated ~~at the following schedule and shall be determined in conformance with the criteria specified in Appendix J of 10CFR50, Appendix J, as modified by approved exemptions.~~ The provisions of Technical Specification 4.0.2 are not applicable to the test intervals specified in 10 CFR 50, Appendix J.

- a. ~~Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at 40 ± 10 month intervals during shutdown at P_1 , 49 psig, or P_2 , 25 psig, during each 10 year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection. (Deleted.)~~
- b. ~~If any periodic Type A test fails to meet either $0.75 L_1$ or $0.75 L_2$, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet $0.75 L_1$ or $0.75 L_2$, a Type A test shall be performed at each plant shutdown for refueling or every 18 months, whichever occurs first, until two consecutive Type A tests meet $0.75 L_1$ or $0.75 L_2$, at which time the above test schedule may be resumed. (Deleted.)~~
- c. ~~The accuracy of each Type A test shall be verified by a supplemental test which: (Deleted.)~~
- ~~1. Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within $0.25 L_1$ or $0.25 L_2$.~~
 - ~~2. Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.~~
 - ~~3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage at P_1 , 49 psig or P_2 , 25 psig.~~

(Page 3/46-3A has been deleted.)

A one time exemption from the accelerated testing requirements of Specification 4.6.1.2.b has been granted. The next Unit 2 Type A test will be performed during the Reload 12 outage after which two consecutive Type A test failures are required to resume the accelerated test schedule.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

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- d. Type B and C tests shall be conducted with gas at P_a , 49 psig, at intervals no greater than 24 months except for tests involving:
 - 1. Air locks.
 - 2. Main steam line isolation valves.
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. Main steam line isolation valves shall be leak tested at least once per 18 months.
- g. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.
- h. The provisions of Specification 4.0.2 are not applicable to 24 month ~~and 40 ± 16 month~~ surveillance intervals.

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ENCLOSURE 7

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
NRC DOCKET NOS. 50-325 AND 50-324
OPERATING LICENSE NOS. DPR-71 AND DPR-62
REQUEST FOR LICENSE AMENDMENTS
CONTAINMENT INTEGRATED LEAKAGE RATE TESTING

TYPED TECHNICAL SPECIFICATION PAGES - UNIT 1

CONTAINMENT SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION (Continued)

- c. The leakage rate to less than or equal to 11.5 scf per hour for any one main steam line isolation valve.

prior to increasing reactor coolant system temperature above 212°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The primary containment leakage rates shall be demonstrated in accordance with the schedule and criteria specified in 10 CFR 50, Appendix J, as modified by approved exemptions. The provisions of Technical Specification 4.0.2 are not applicable to the test intervals specified in 10 CFR 50, Appendix J.

- a. Deleted.
- b. Deleted.
- c. Deleted.
- d. Type B and C tests shall be conducted with gas at P_a , 49 psig, at intervals no greater than 24 months except for tests involving:
 - 1. Air locks.
 - 2. Main steam line isolation valves.
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. Main steam line isolation valves shall be leak tested at least once per 18 months.
- g. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.
- h. The provisions of Specification 4.0.2 are not applicable to 24 month surveillance intervals.

(Pages 3/4 6-3A and 3/4 6-3B have been deleted.)

ENCLOSURE 8

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
NRC DOCKET NOS. 50-325 AND 50-324
OPERATING LICENSE NOS. DPR-71 AND DPR-62
REQUEST FOR LICENSE AMENDMENTS
CONTAINMENT INTEGRATED LEAKAGE RATE TESTING

TYPED TECHNICAL SPECIFICATION PAGES - UNIT 2

CONTAINMENT SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

ACTION (Continued)

- c. The leakage rate to less than or equal to 11.5 scf per hour for any one main steam line isolation valve,

prior to increasing reactor coolant system temperature above 212°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The primary containment leakage rates shall be demonstrated in accordance with the schedule and criteria specified in 10 CFR 50, Appendix J, as modified by approved exemptions. The provisions of Technical Specification 4.0.2 are not applicable to the test intervals specified in 10 CFR 50, Appendix J.

- a. Deleted.
- b. Deleted.
- c. Deleted.
- d. Type B and C tests shall be conducted with gas at P_s , 49 psig, at intervals no greater than 24 months except for tests involving:
 - 1. Air locks,
 - 2. Main steam line isolation valves,
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. Main steam line isolation valves shall be leak tested at least once per 18 months.
- g. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.
- h. The provisions of Specification 4.0.2 are not applicable to 24 month surveillance intervals.

(Page 3/4 6-3A has been deleted.)